Karlsruhe Institute of Technology

Validation calculations for Serpent full-core pin-by-pin burnup in Light Water Reactors

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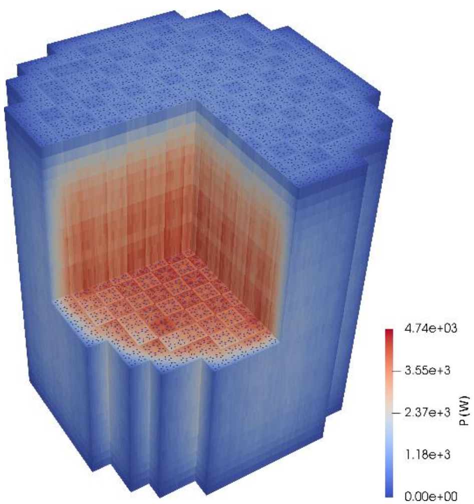
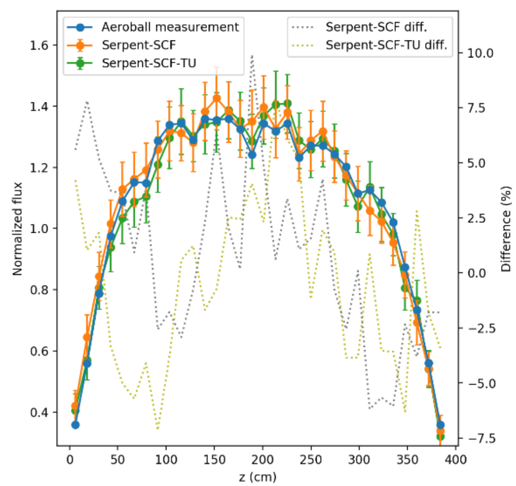
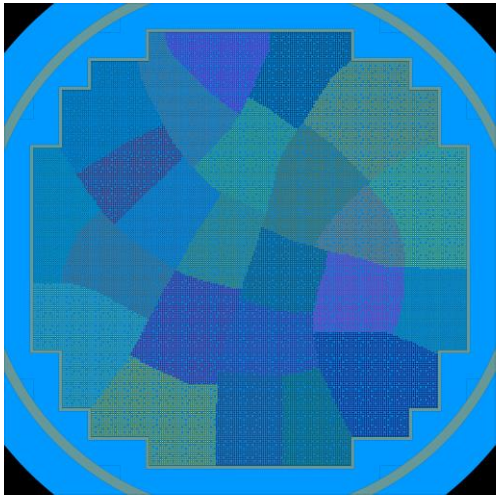




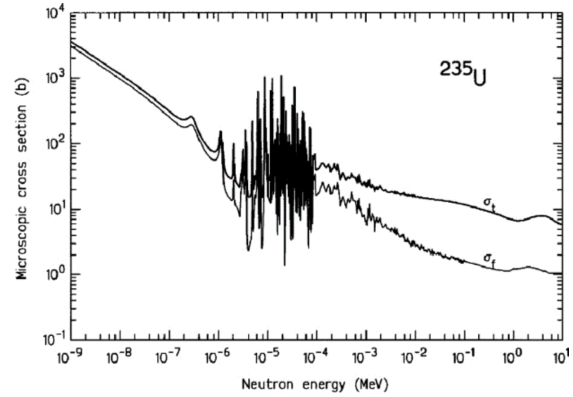


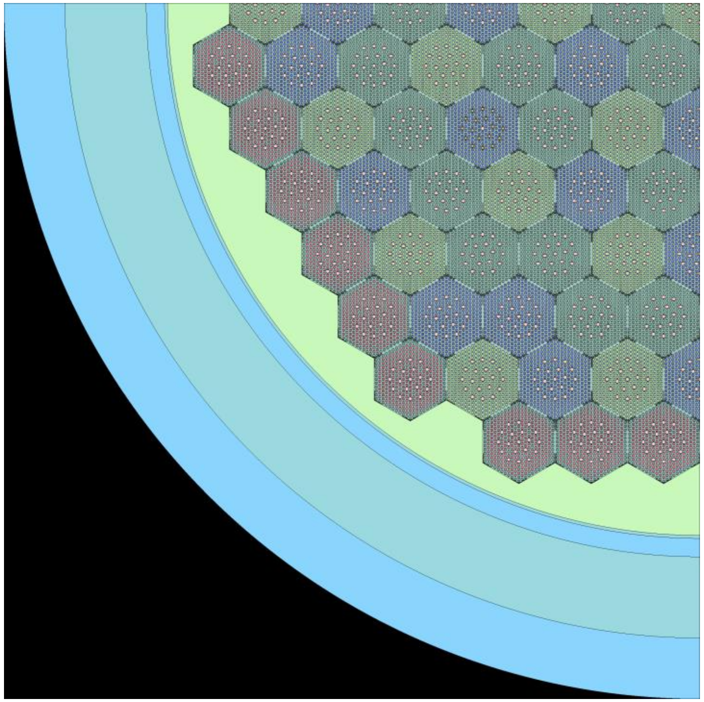
Introduction

- Burnup analysis for Light Water Reactors (McSAFE project):
  - Multiphysics:
    - Monte Carlo neutron transport (criticality, power, burnup).
    - Subchannel thermalhydraulics (cooling conditions, safety parameters).
    - Fuel-performance analysis (thermomechanics, burnup, safety parameters).
  - High-fidelity methods:
    - Less approximations compared to industry standards.
    - Higher resolution (geometry, neutron direction and energy).
  - High Performance Computing (HPC).
- Proposed methodology:
  - Serpent-SCF-TU full-core pin-by-pin depletion.
  - Objectives:
    - Burnup calculation with fuel-performance analysis for realistic core states.
    - Potential improvement of the calculation of local safety parameters.
    - Proof of concept for industry-like (?) applications.
  - Validation for PWR and VVER reactors using experimental data.

Monte Carlo neutron transport (Serpent 2)

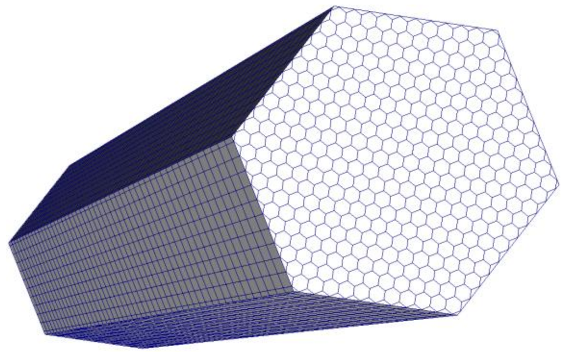
- Neutron transport calculation:
  - Stochastic (not deterministic) method.
  - No inherent approximations:
    - Continuous-energy nuclear data.
    - Arbitrary level of geometrical detail.
    - Explicit angular treatment.
  - Large calculation times.
- Depletion calculation:
  - Direct calculation.
  - ~1700 nuclides.
  - Huge memory demand.
- Modelling approach:
  - Full-core pin-by-pin geometry.
  - Pin-level thermalhydraulic feedback.
  - Pin-by-pin burnup.

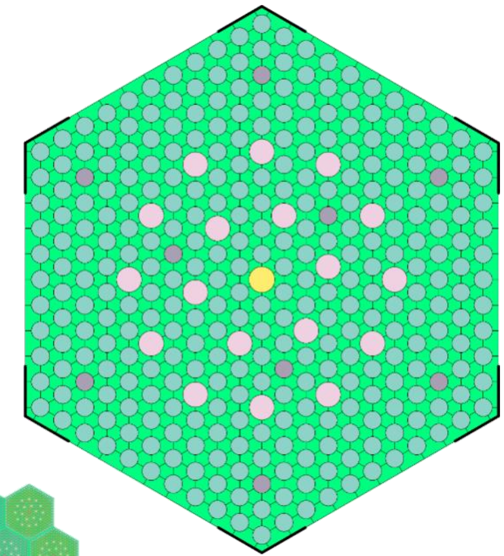


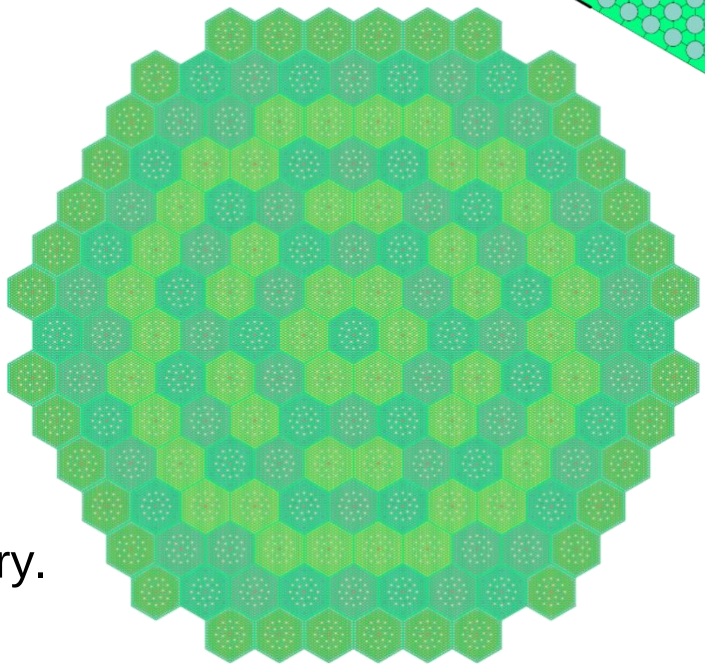


Subchannel thermalhydraulics (SUBCHANFLOW)

- Coolant flow calculation:
  - Subchannel approach.
  - Turbulent mixing.
  - Empirical correlations.
- Fuel rod calculation:
  - Radial scheme.
  - Simple fuel-clad gap model.
  - Simple thermomechanics.
- Safety parameters:
  - DNBR.
  - Fuel temperature.
  - Clad temperature.
- Modelling approach:
  - Full-core pin-by-pin geometry.
  - Subchannel-level model.

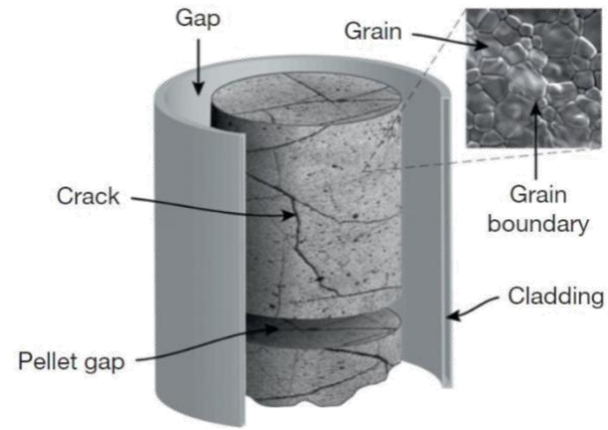


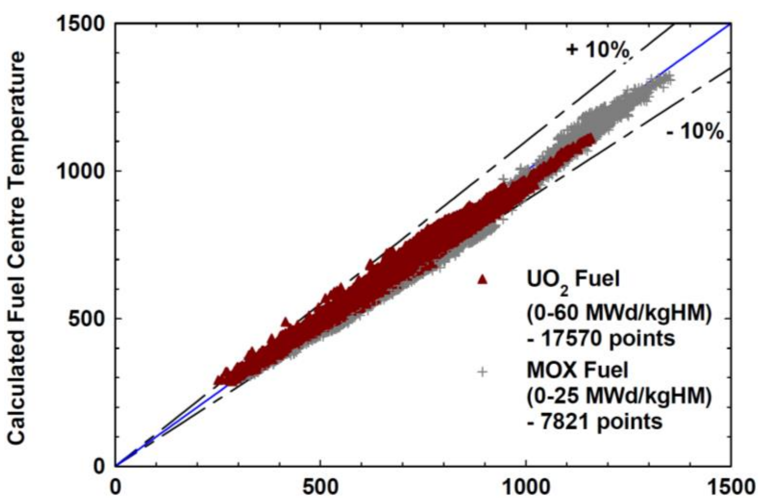


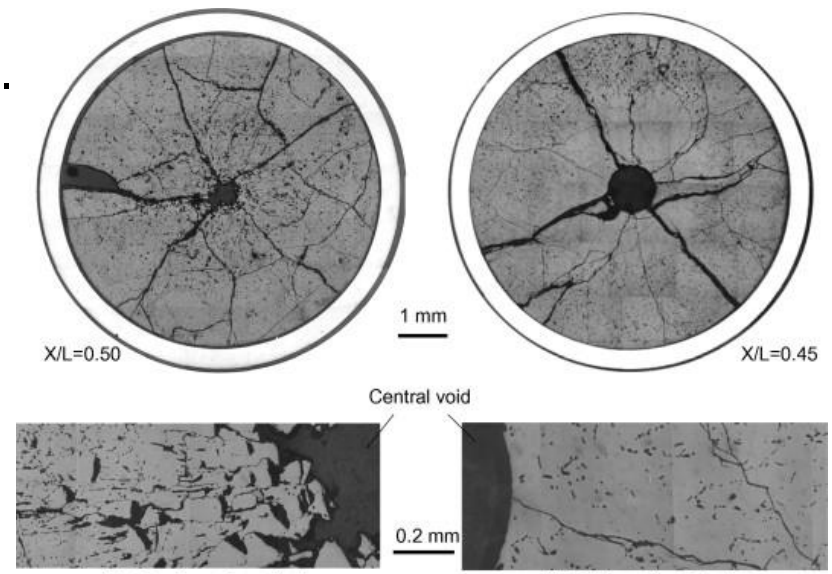


Fuel-performance analysis (TRANSURANUS)

- Main thermomechanical phenomena:
  - Fuel-cladding gap conductance and width.
  - Fission gas release, filling gap pressure.
  - Pellet-cladding mechanical interaction.
  - Thermal and irradiation-induced densification.
  - Swelling, creep, plasticity, cracking, relocation.
- Modelling approach:
  - Fuel-performance analysis of all rods.



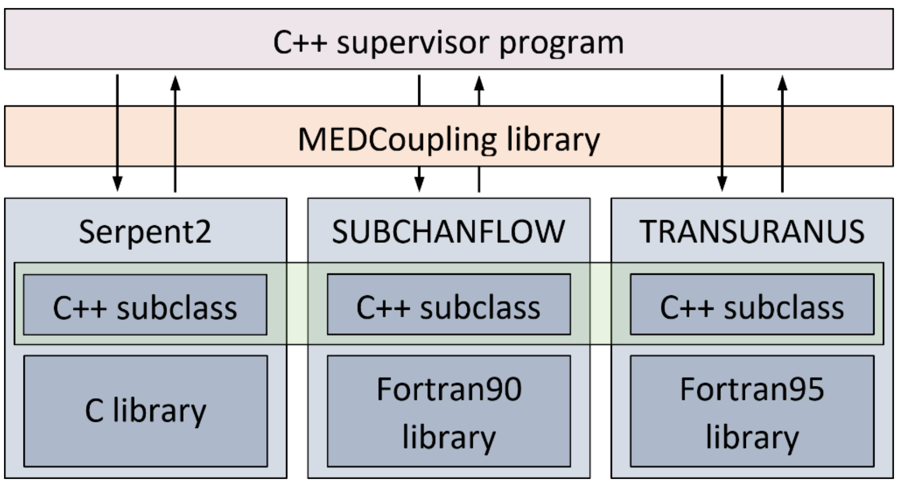


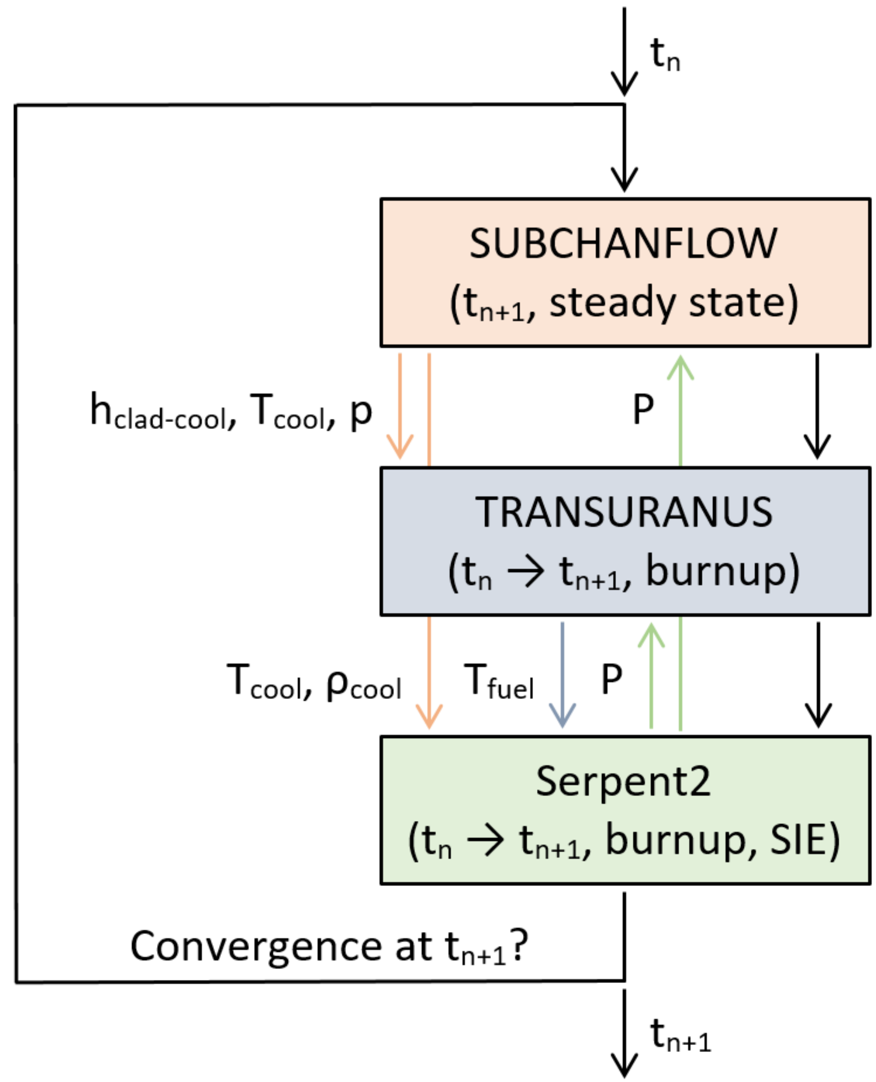


(a) 10-minute irradiation (b) 24-hour irradiation

Coupled calculation scheme for depletion

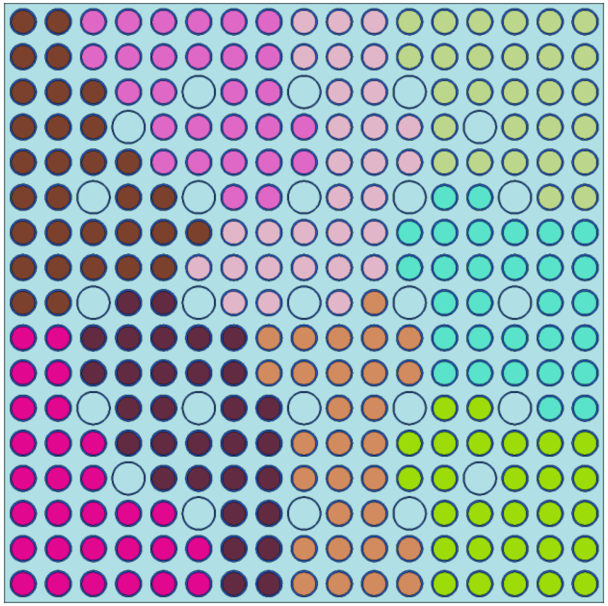
- Semi-implicit (?) iterative scheme.
- Pin-level feedback.
- Burnup in Serpent and TU.
- Potential improvements in TU:
  - Radial power distributions.
  - Fast flux feedback.
  - Serpent burnup calculation.





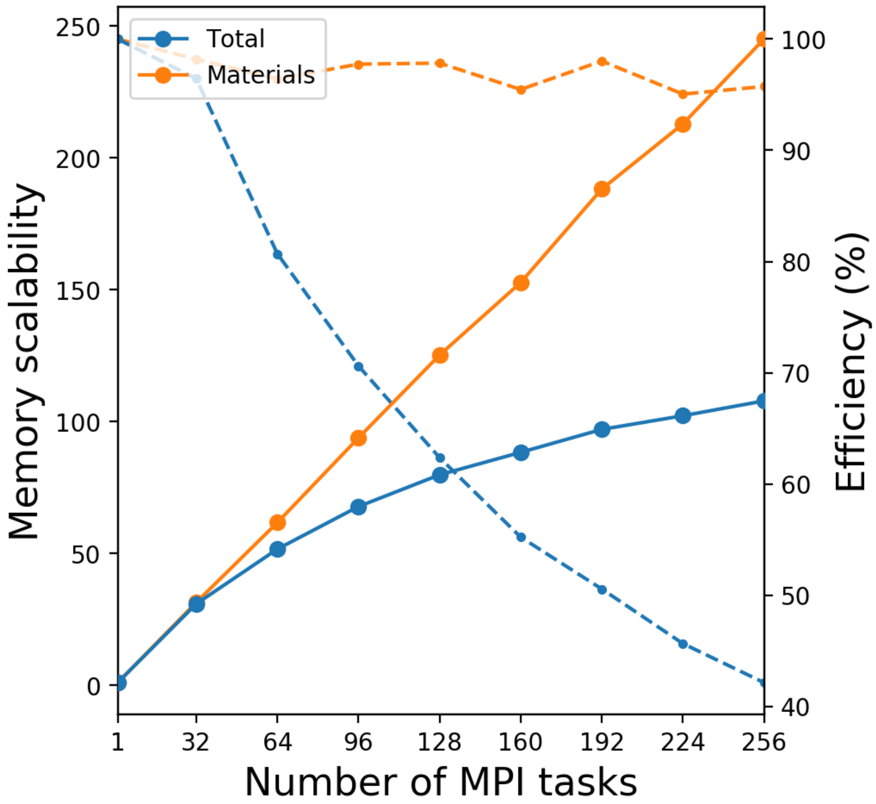
Collision-based Domain Decomposition

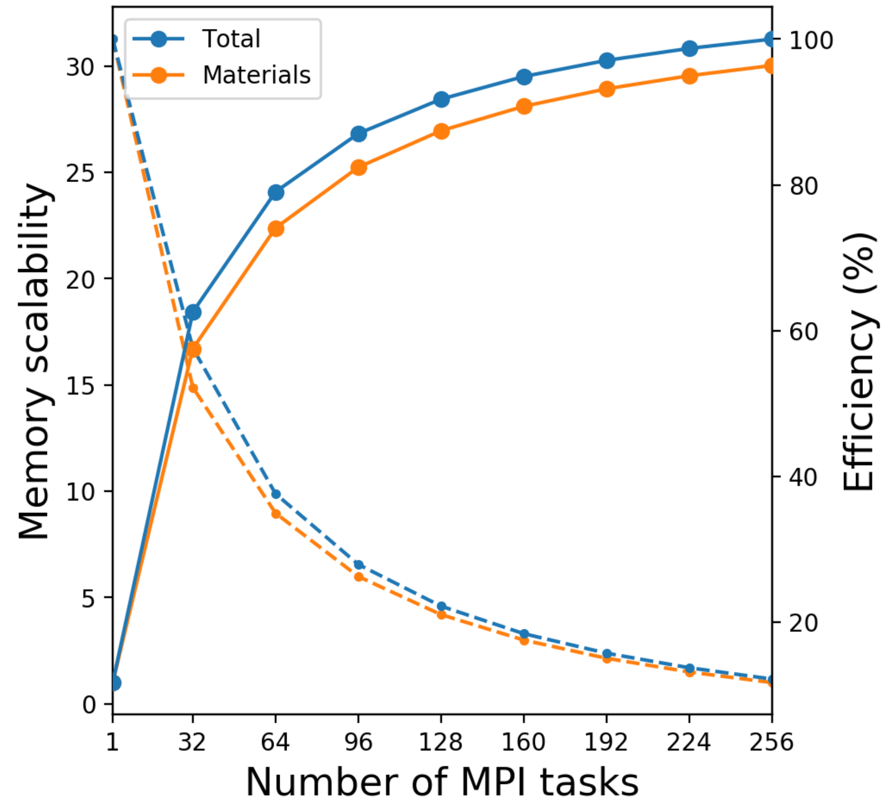
- Full-core pin-by-pin burnup:
  - Hugh runtimes (~10^9 neutron histories per transport calculation).
  - Massive memory demand (~TB, larger than the in-node memory in HPC).
- Collision-based Domain Decomposition (CDD):
  - Memory scalability in burnup calculations:
    - Burnup materials decomposed in domains (MPI tasks).
    - All other data replicated across domains.
  - Tracking scheme:
    - Particle transfers across domains.
    - Asynchronous MPI communications.
    - Optimized tracking termination control.
    - No physical or numerical approximations.



CDD performance in a PWR-like system

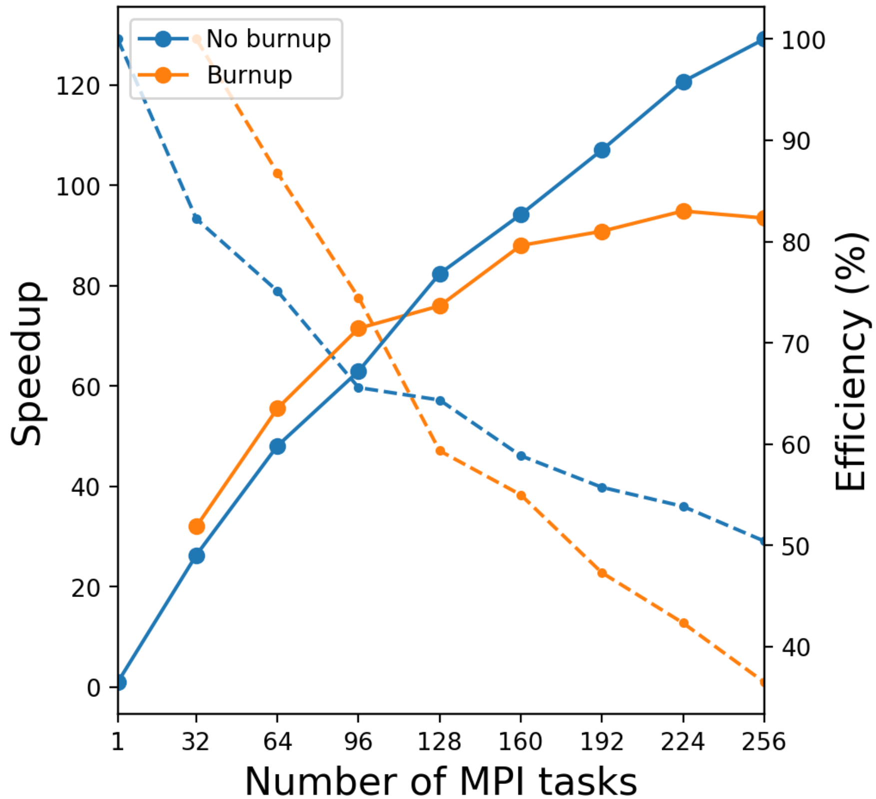
- Memory scalability:
  - Tests done in the ForHLR II cluster (KIT/SCC) up to 5,120 cores.
  - Much better scalability for opti mode 4 (left) than for mode 2 (right).

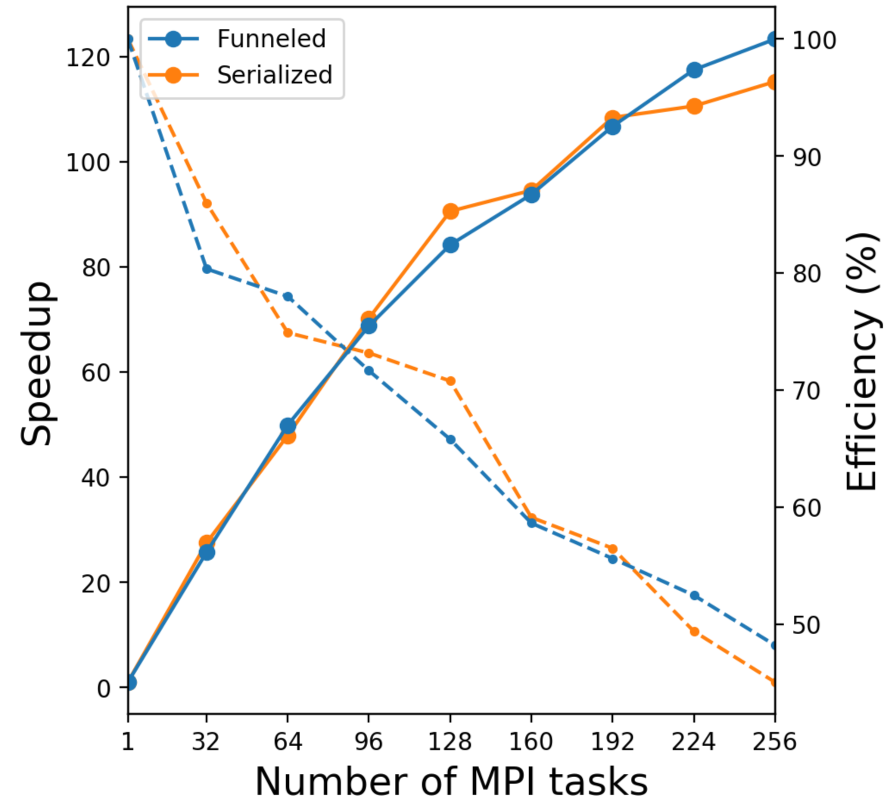




CDD performance in a PWR-like system

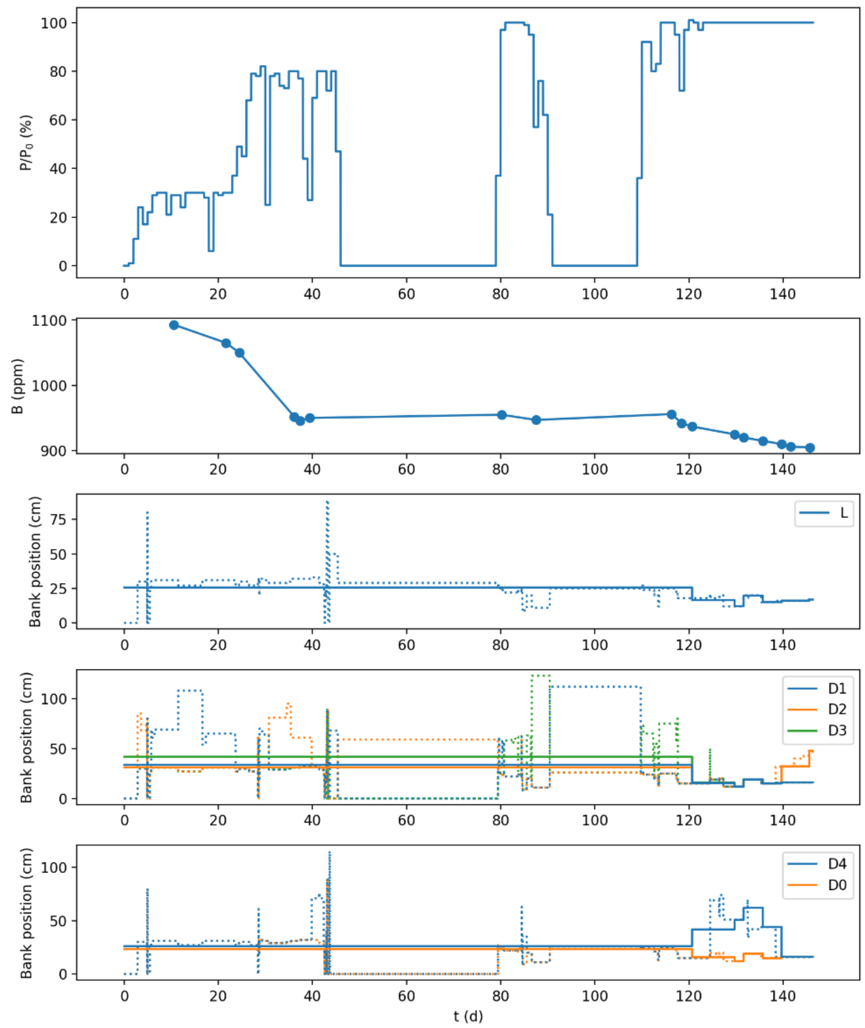
- Speedup (~10^6 particles per cycle):
  - Tests done in the ForHLR II cluster (KIT/SCC) up to 5,120 cores.
  - Acceptable speedup, some penalty from CDD.





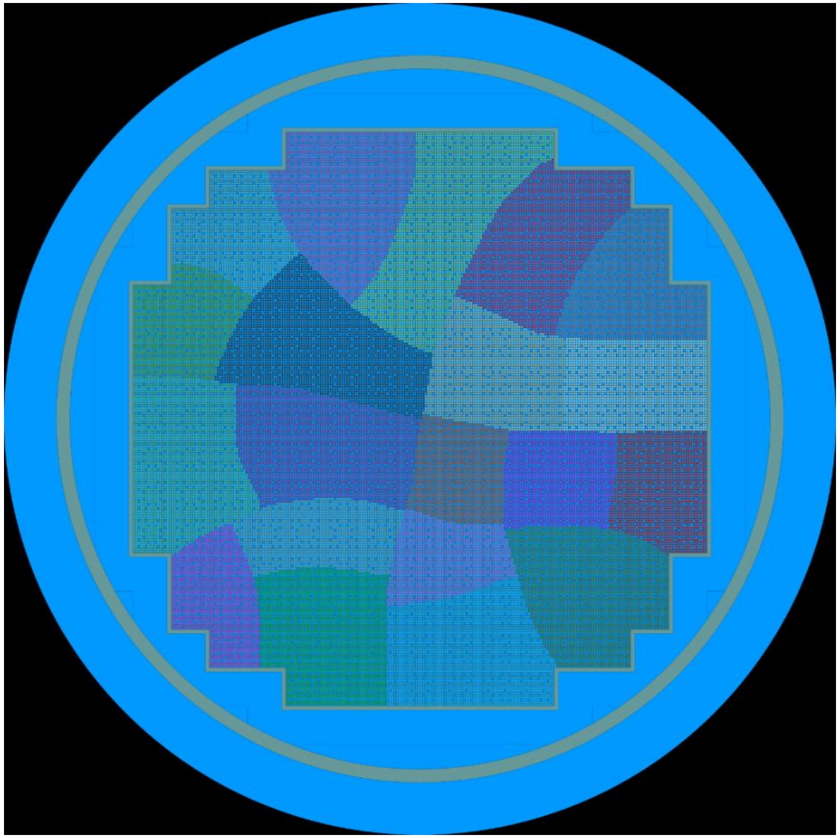
Validation for a Pre-Konvoi PWR reactor

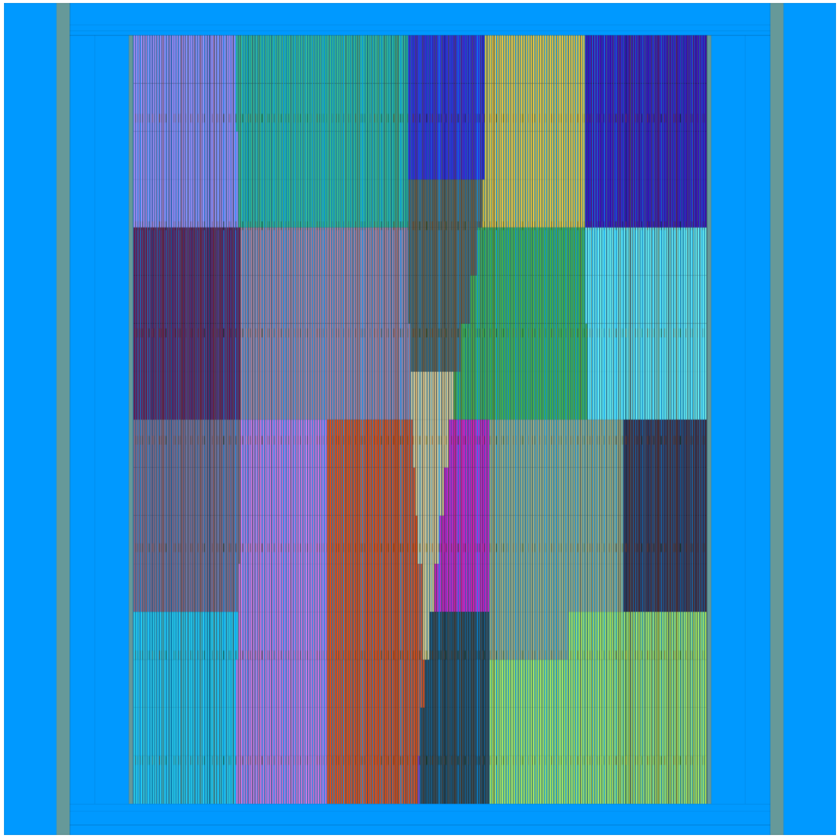
- Burnup history:
  - Nominal power (3765MW).
  - Measured point:
    - 64.9 effective power days.
    - Full power.
    - Equilibrium xenon.
    - Experimental data:
      - Critical boron concentration.
      - Neutron flux (aeroballs).
  - Critical boron iteration.
  - Control rod movements:
    - Constant for each burnup step.
    - Power-weighted averaged.
- HPC resources (ForHLR II):
  - 64 nodes, 20 CPU/node.
  - 7 day runtime limit.



Material decomposition for CDD

- Material decomposition:
  - 728,768 burnable materials, 64 nodes, 68GB per node, opti mode 1.
  - Graph-based method using Metis (not available in the official release).





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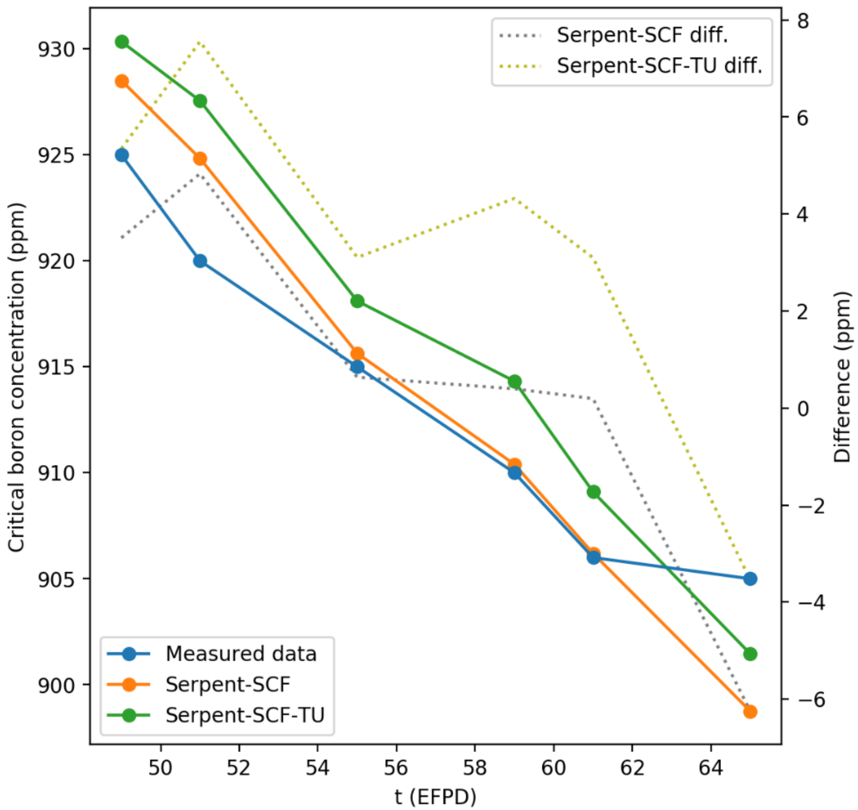
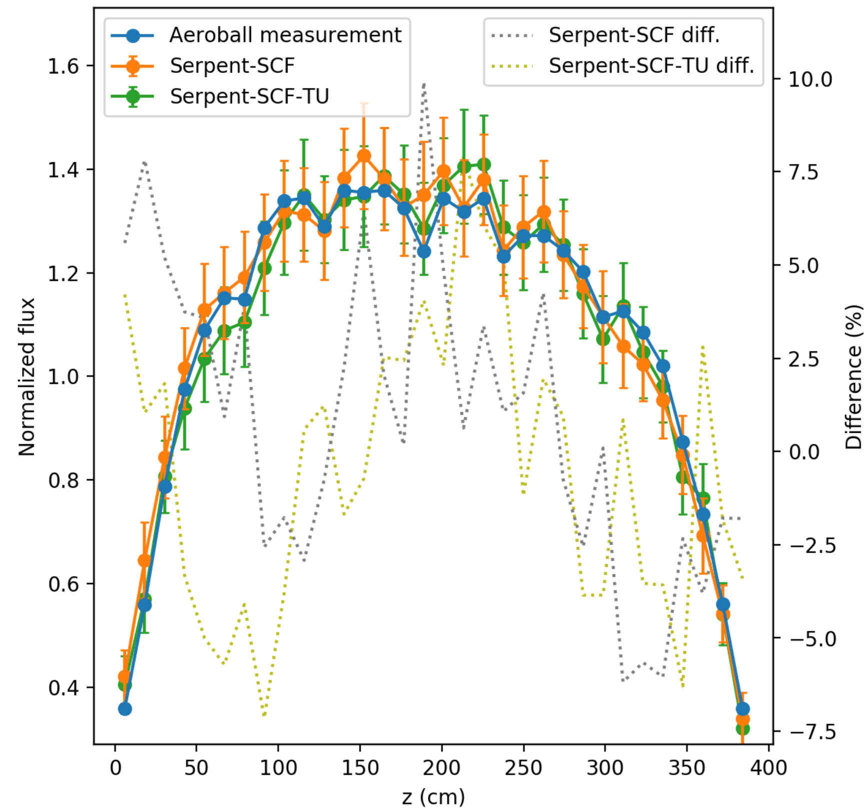
Validation results

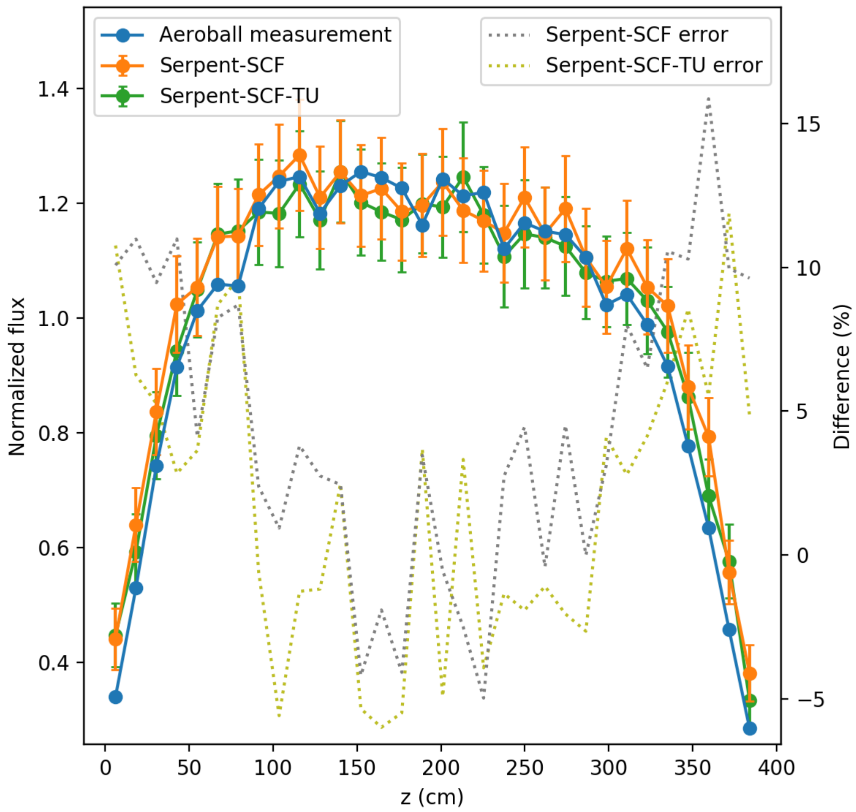
- Good agreement between results and experimental measurements.
  - Differences in critical boron concentration within a few ppm.
  - Aeroball neutron flux profiles within the statistical range of the results.

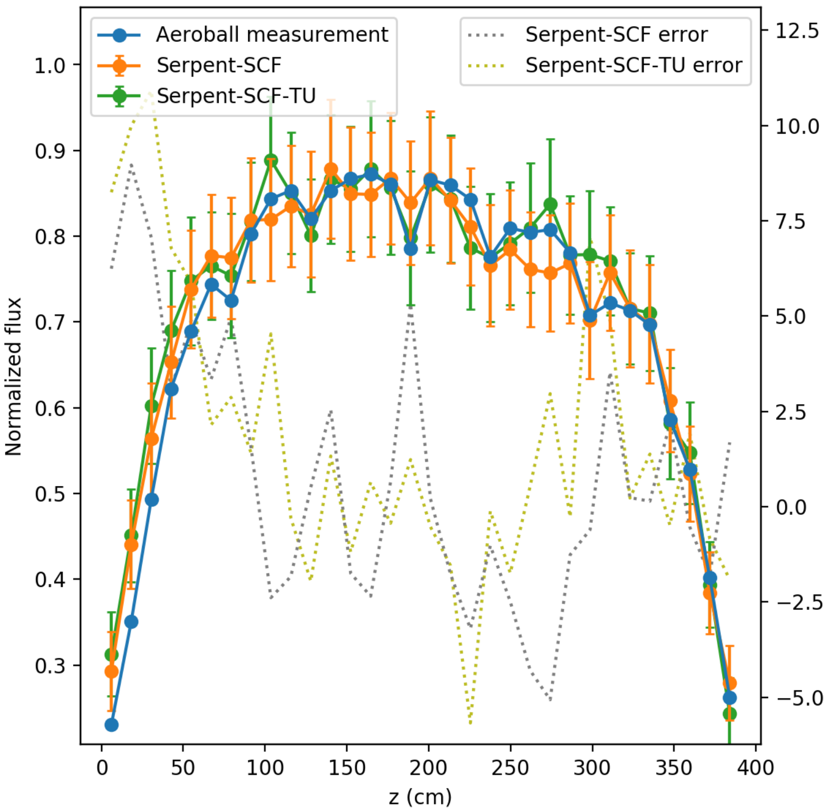




Validation results

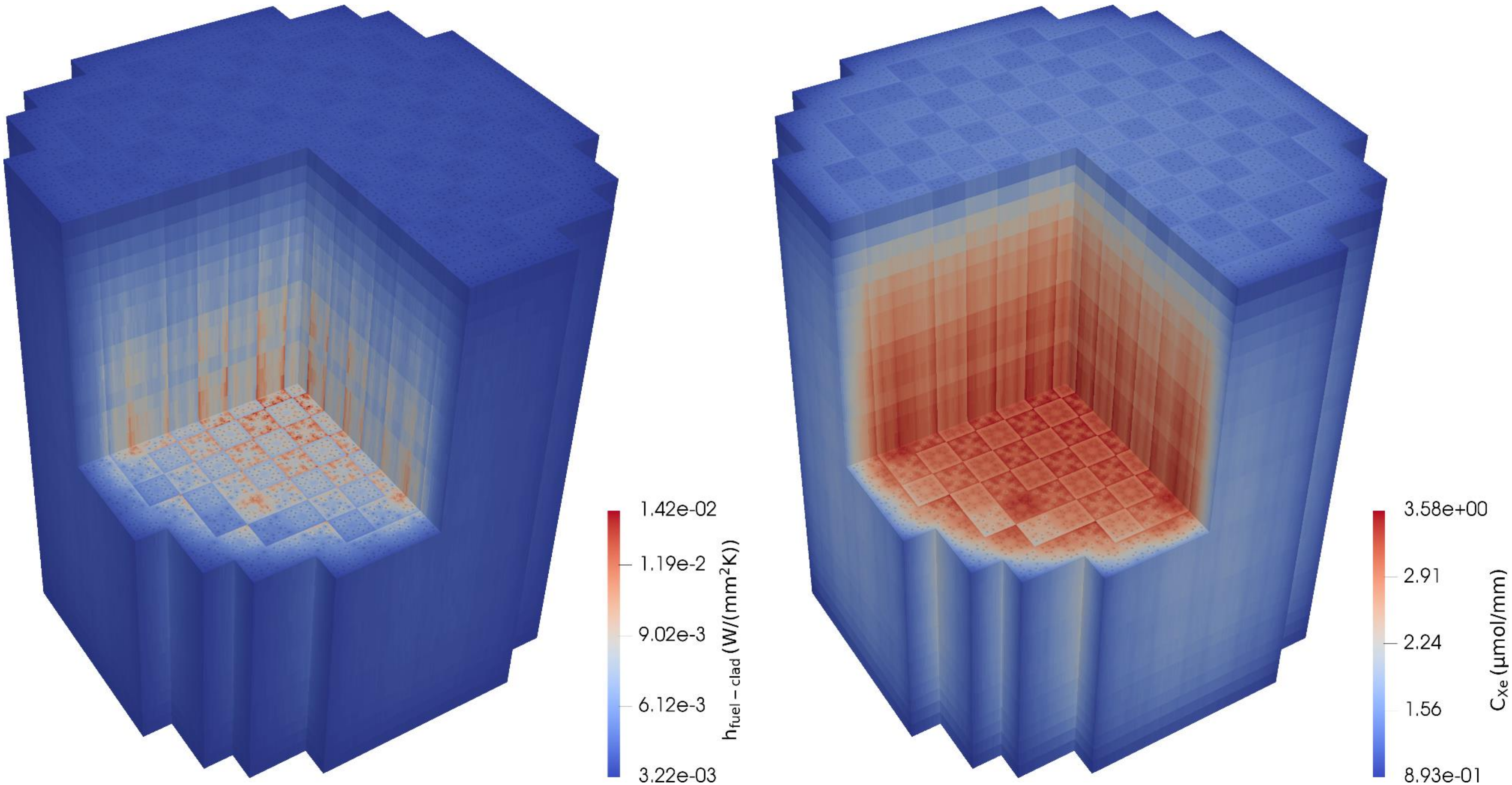
- Some more aeroball results (28 in total):





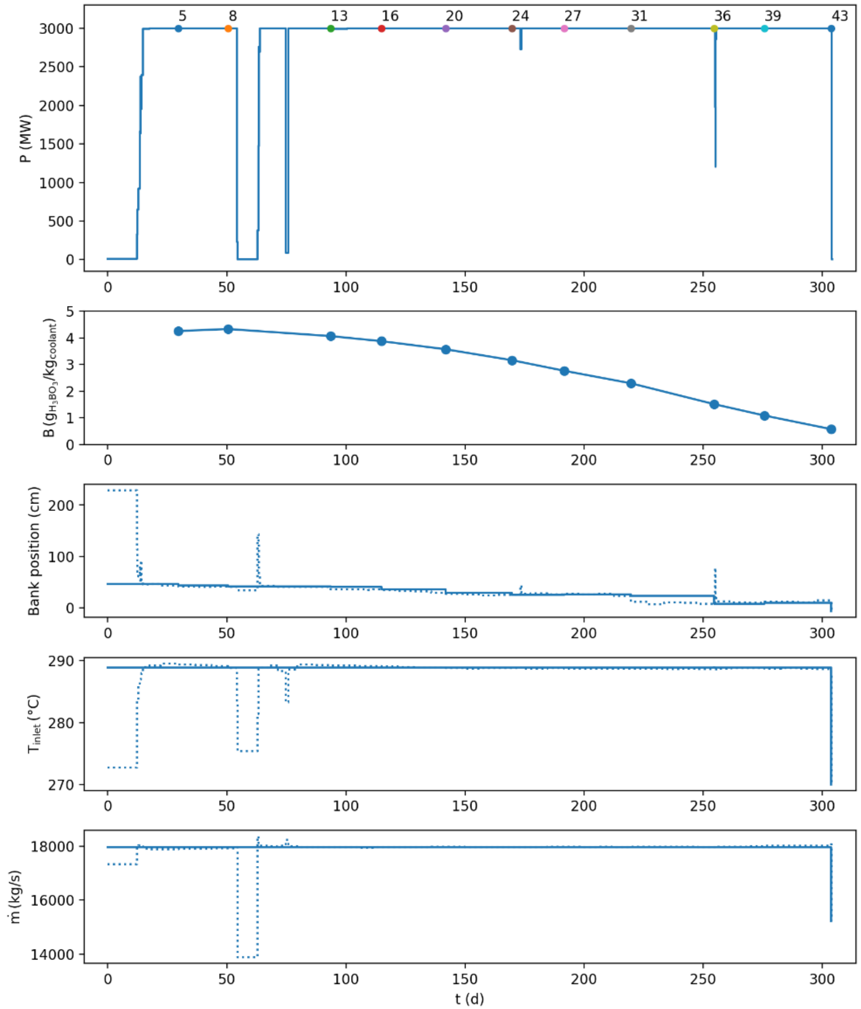
Fuel-performance capabilities

- Fuel-cladding gap conductivity and Xe release at 2.35 MWd/kg:



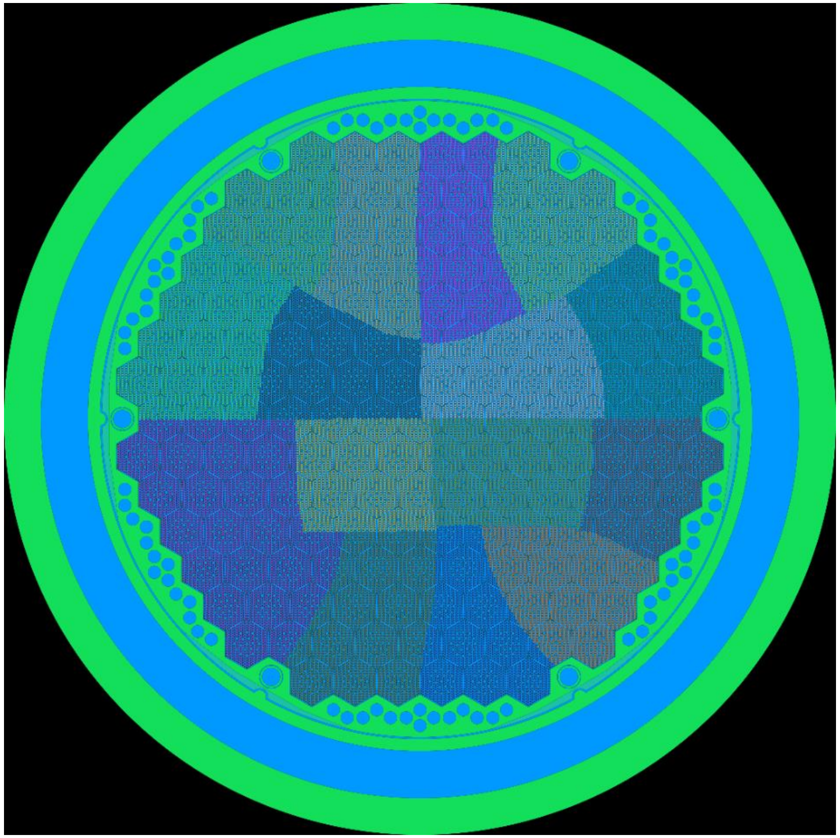
Validation for a VVER-1000 reactor

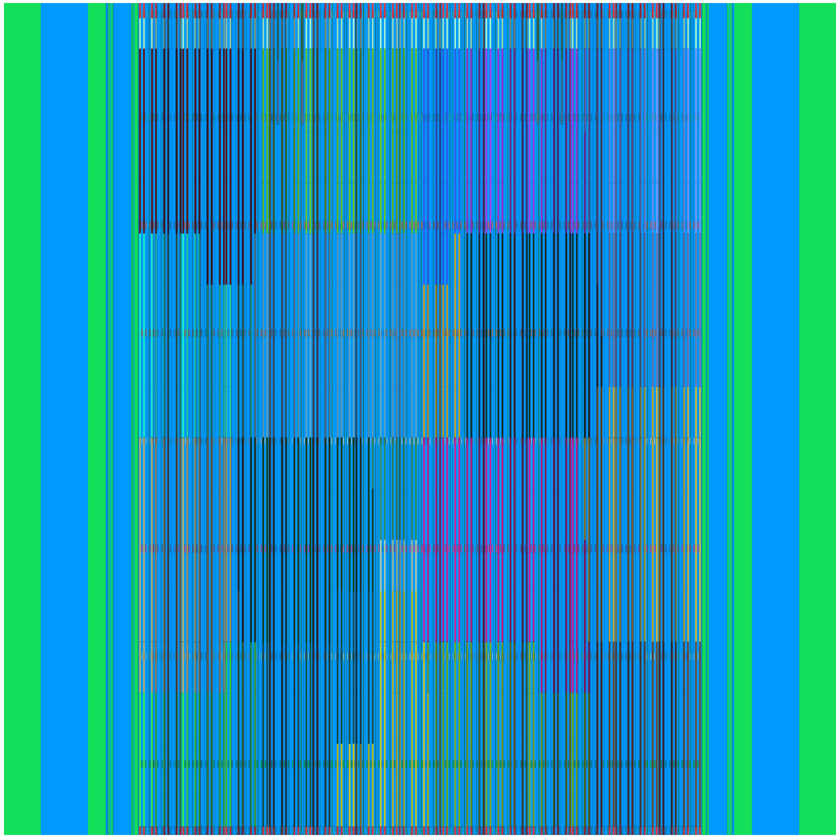
- Burnup history:
  - Nominal power (3000MW).
  - Measured points:
    - Full power.
    - Equilibrium xenon.
    - Experimental data:
      - Critical boron concentration.
      - Power (CMS).
      - Neutron flux (SPNDs).
  - Critical boron iteration.
  - Control rod movements:
    - Constant for each burnup step.
    - Power-weighted averaged.
- HPC resources (ForHLR II):
  - 64 nodes, 20 CPU/node.
  - 7 day runtime limit.



Material decomposition for CDD

- Material decomposition:
  - 813,696 burnable materials, 64 nodes, 68GB per node, opti mode 1.
  - Graph-based method using Metis (not available in the official release).





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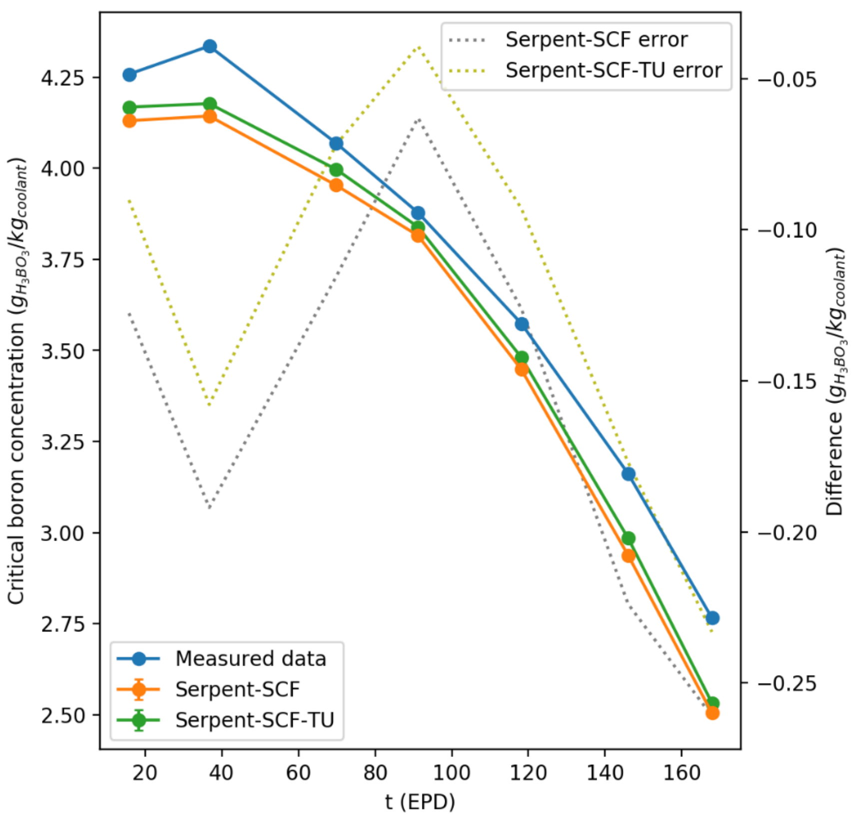
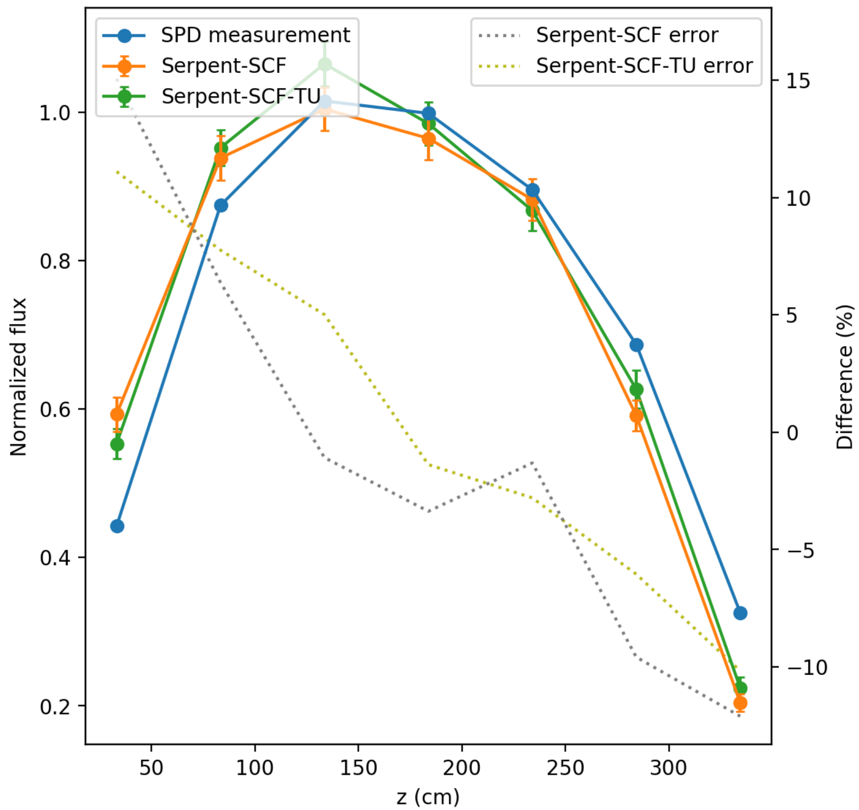
Validation results

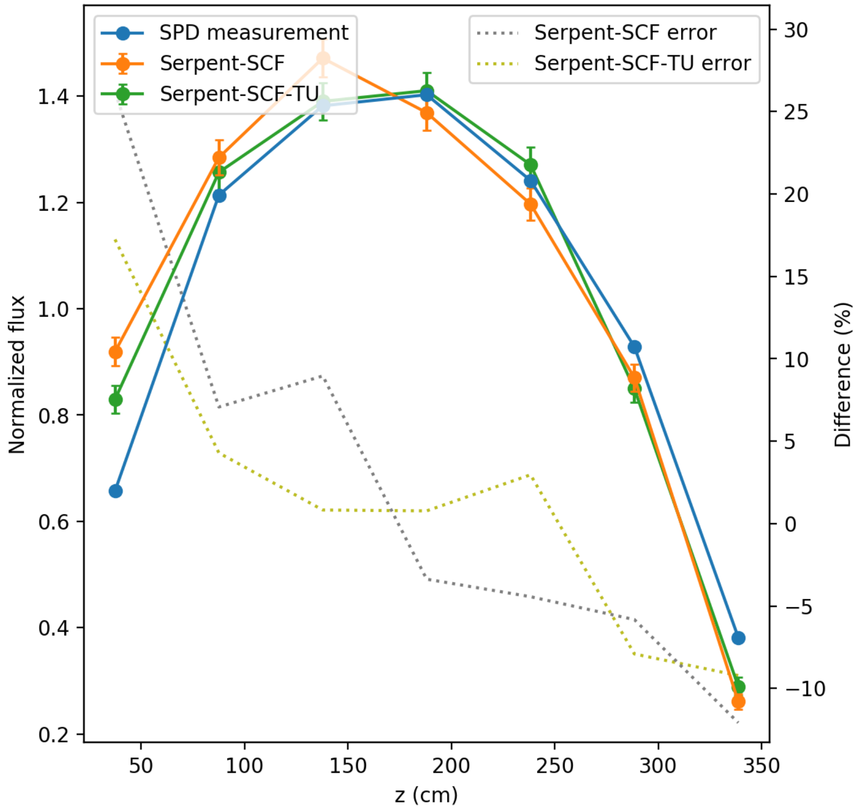
- Good agreement between results and experimental measurements.
  - Differences in critical boron concentration within a few ppm.
  - Some good SPD results, some bad (some SPDs are up to 70% burnt).

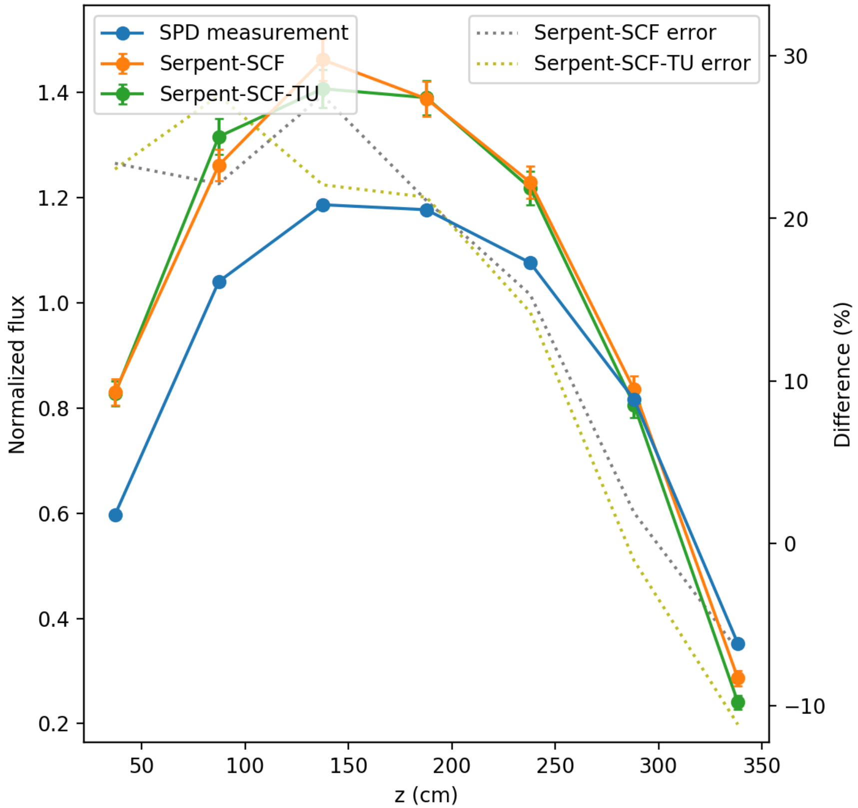




Validation results

- Some more SPD results (64 in total):





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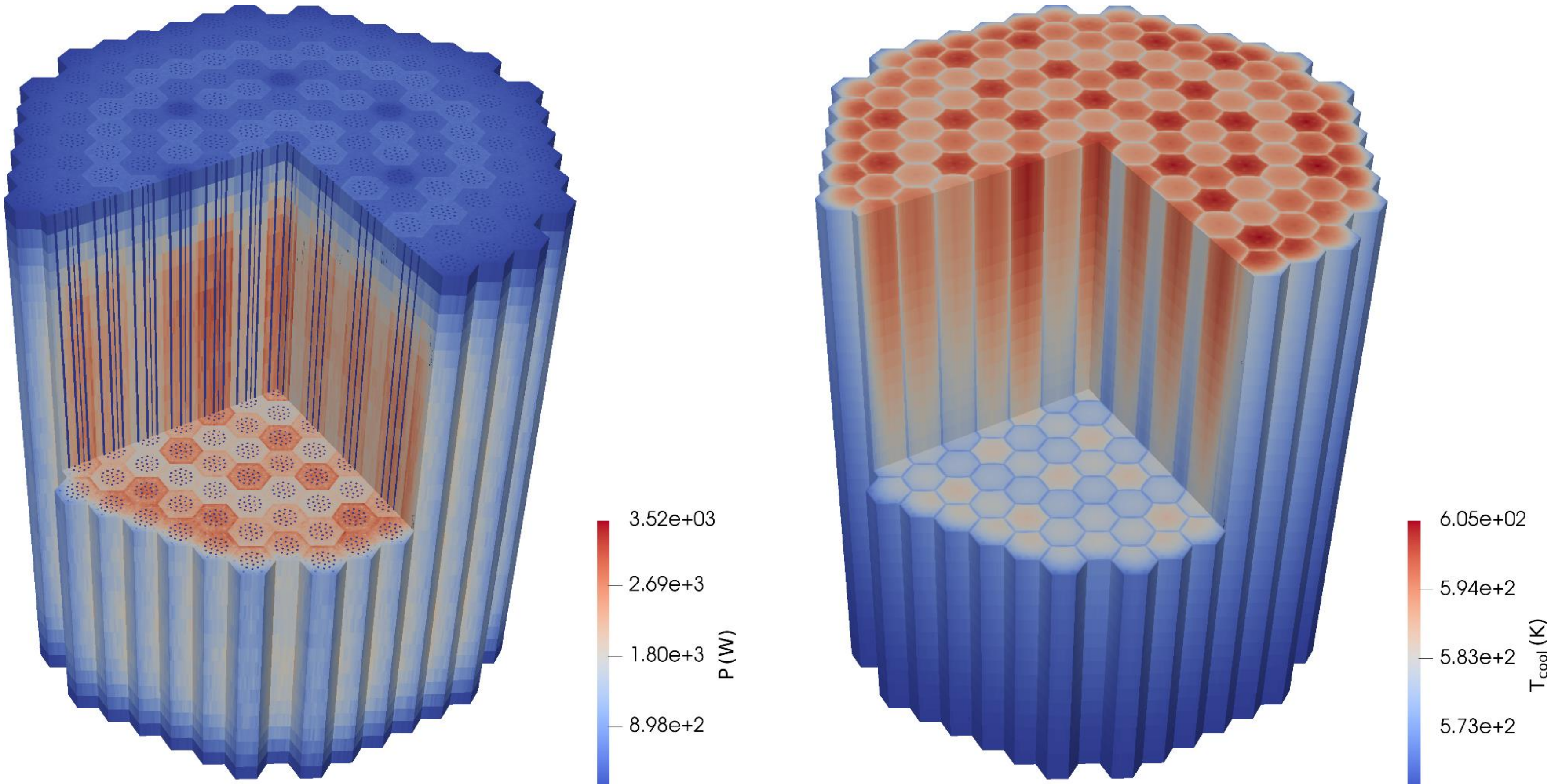
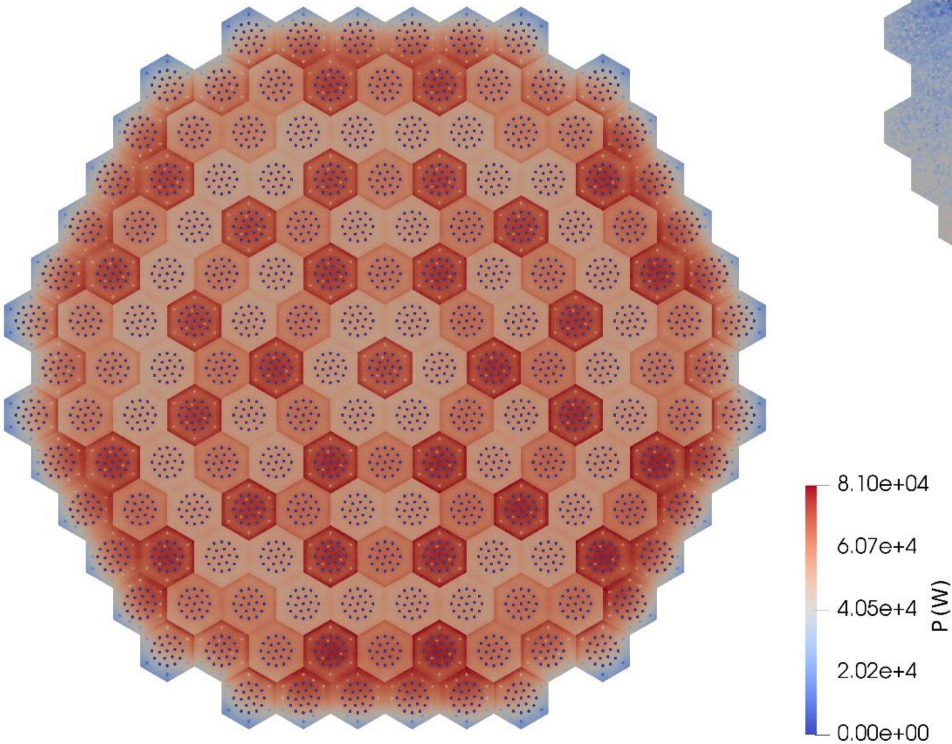
Power and coolant temperature (EOC)

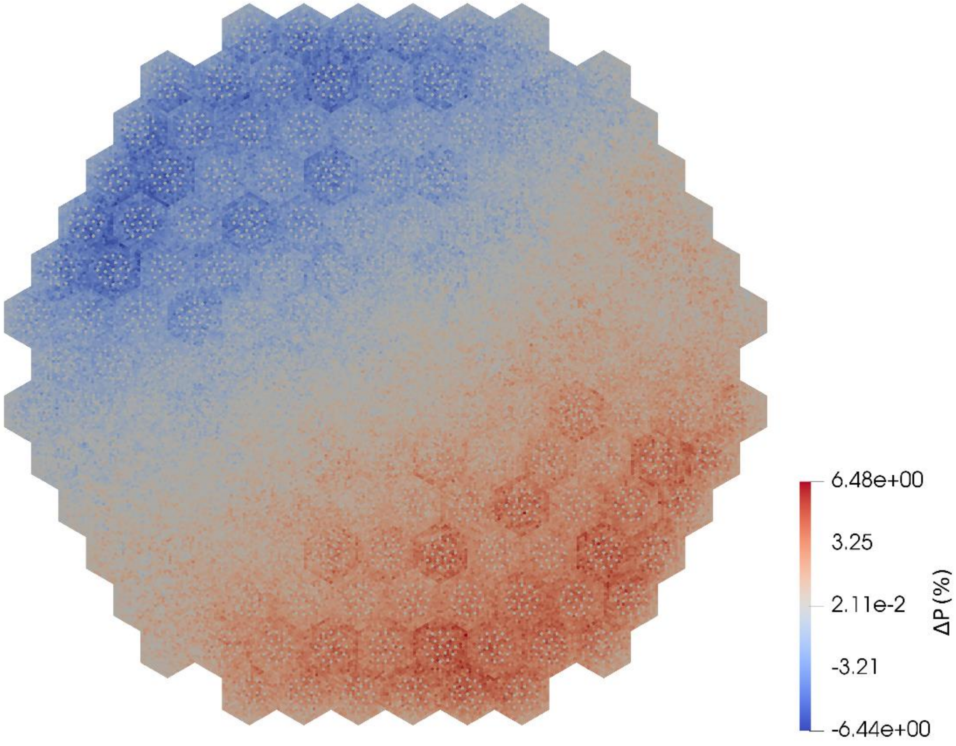
- Serpent-SCF-TU solution:



Power oscilations (EOC)

- Serpent-SCF-TU solution:





Conclusions

- Developed calculation system:
  - Coupled neutronic-thermalhydraulic-thermomechanic depletion scheme.
  - CDD method for memory scalability.
  - Full-core pin-by-pin burnup capabilities.
- Validation:
  - Pre-Konvoi PWR up to 65 EFPD (done).
  - VVER-1000 up to 280 EFPD (more challenging due to calculation time).
  - Runtime up to 7 days in 64 computing nodes (1,280 cores).
- Fully coupled burnup calculations:
  - Improvement in the modelling of the fuel during irradiation.
  - Minor impact on the neutronic solution.
  - Large impact on fuel parameters such as gap behavior and fuel temperature.

Questions?


THANK
YOU
GRACIAS
ARIGATO
SHUKURIA
DANKSCHEEN
TASHAKKUR ATU
YAQHANYELAY
SUKSAMA
EKHMET
BİYAN
SHUKRIA
TINGKI
GRAZIE
MEHRBANI
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