

KIT Neutronic Computational Tools for SMR-Core Analysis

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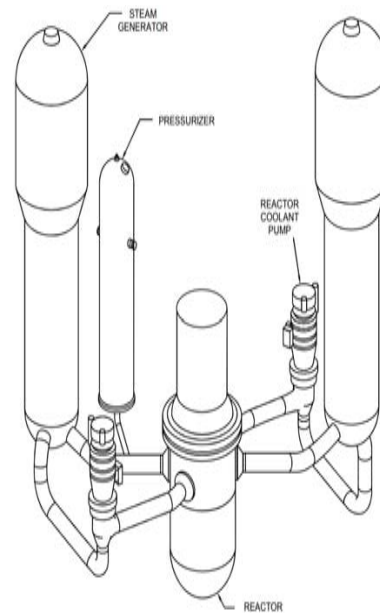


Content

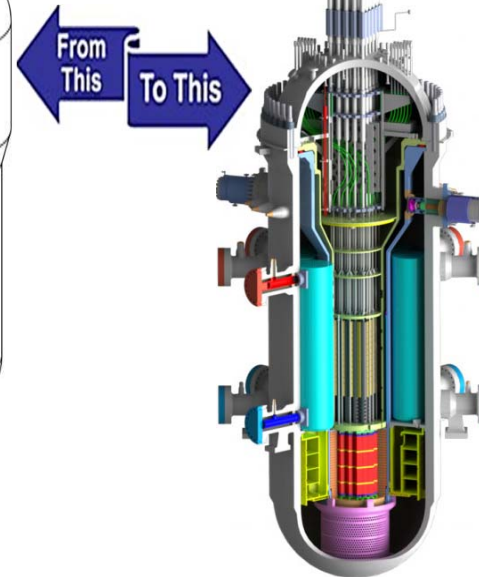
- Background
- Challenges
 - Neutronics
 - Thermal hydraulics
- KIT approach
- Outlook

Background [1]

- Increase interest on SMR
 - Water cooled
 - Gas-cooled
 - Liquid-metal cooled
 - Etc.
- Multiple-use
 - Electricity, heat
 - Water desalination
 - Hydrogen production
- Attractiveness:
 - Module factory fabrication
 - Pursuing economies of series production and
 - Short construction times



PWR

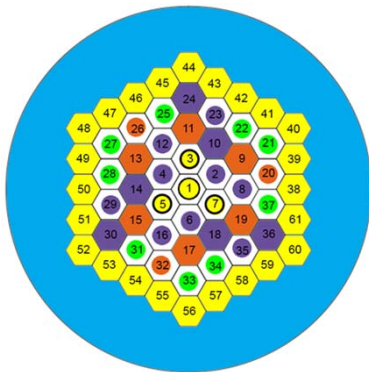


SMR
(SMART)

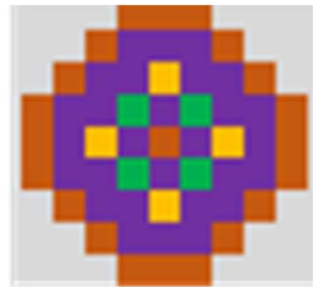
SMR will be build only if economically competitive and safe

Water Cooled SMR-Cores: Different designs [1,2]

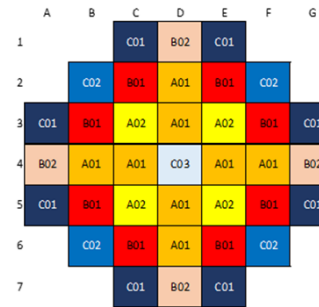
CAREM-Like core



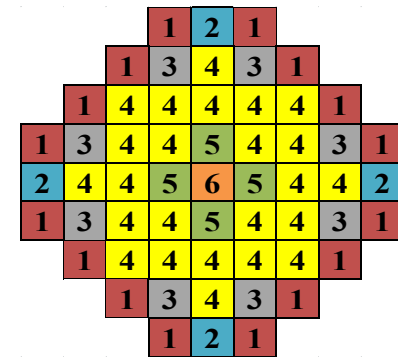
FSMR Core



NuScale

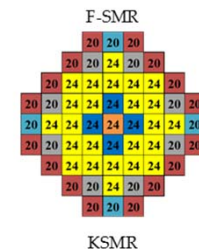
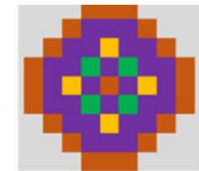


KSMR



Common Features of Water-cooled SMRs [1]

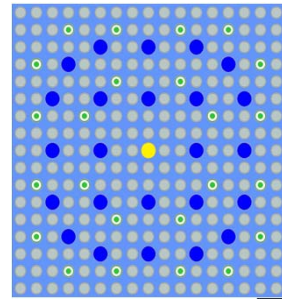
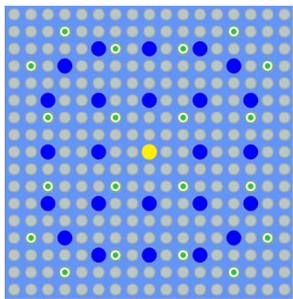
- Compactness
 - Small size (H and D)
 - Heterogeneity (radial, **axial**)
 - High leakage
 - Harder spectrum
 - Complex control rod designs
 - Different types
 - **Axial heterogeneity**
 - Increased role of reflector
- Boron free cores:
 - Need innovative control rod design
 - Optimized shutdown reactivity
 - Reduced reactivity swing over the cycle
 - Etc.



→ Innovations needed to improve economics and keep high safety

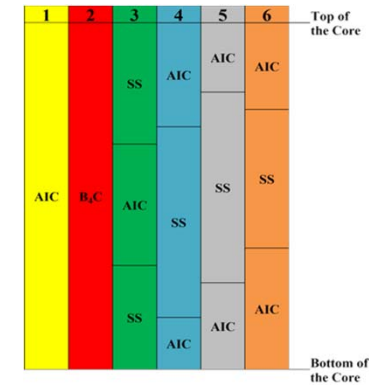
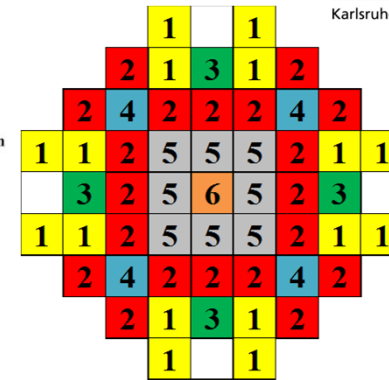
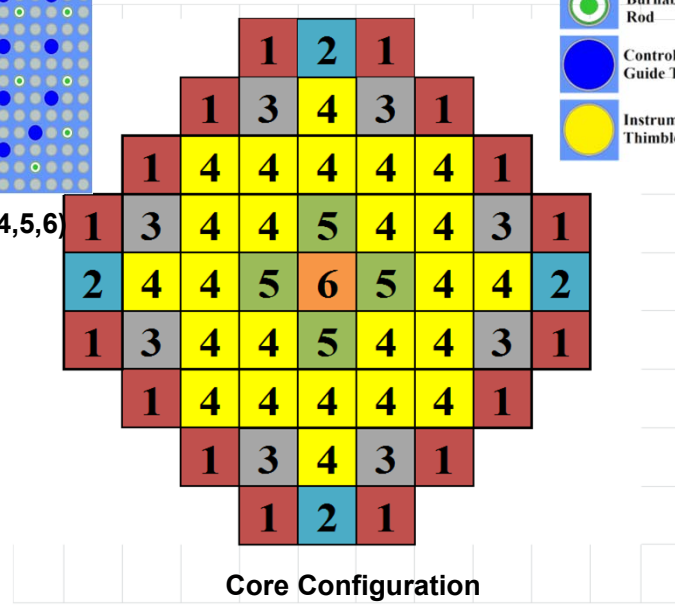
KSMR Design: Example [3,4,5]

Highly heterogeneous /complex control rods



With 20 BPRs (1,2,3) With 24 BPRs (4,5,6)

FA-1	FA-2	FA-3	FA-4	FA-5	FA-6	Top of the Core
1.1		3.1	4.1	5.1	6.1	
1.2		3.2	4.2			
1.3	2.1		4.3	5.2	6.2	
		3.3				
1.4						
1.5		3.4	4.4	5.3	6.3	Bottom of the Core



Ref. [3,4,5]

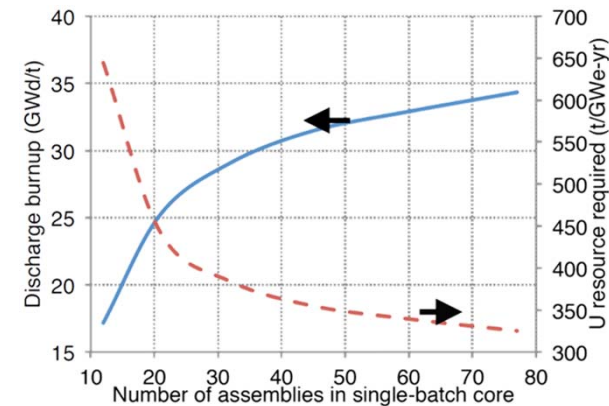
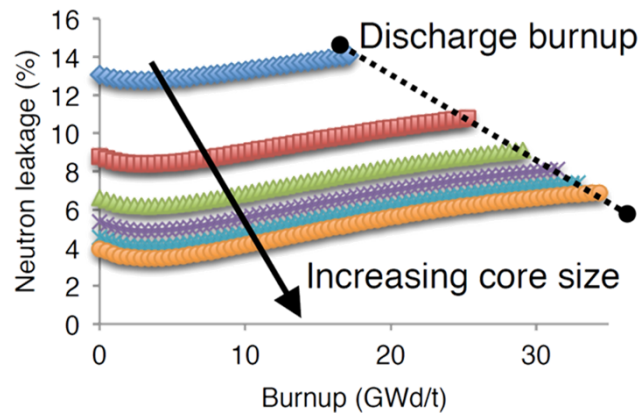
Challenges

Neutronics: Core Design Challenges [30]

- Typical SMR core size is 37 – 89 FAs
 - NuScale: 37 FAs, SMART: 57 FAs,
 - CAREM-25: 61 FAs
 - mPower: 69 FAs
- Core height is almost half of large LWR

- Increased neutron leakage
- Less degree of freedom

Lower fuel utilization



Ref [30]

Safety-related Thermal Hydraulics [1] :

- Experimental data exist but proprietary (SMR developers)
- Public SMR-specific data for research community needed e.g.
 - Cross flow in the core
 - Helical HX
 - Transition from
 - Forced to natural convection
 - Natural to forced convection
 - Safety parameters like
 - CHF
 - 3D flow inside the RPV
 - Effectiveness of PRHRS
 - Stability of natural convection flow
- Data need for code validation



McSAFER Solution Approach:

- COSMOS-H experimental program:
 - Fundamental HT, boiling, CHF
- HWAT experimental program`:
 - System behavior under natural circulation
 - Transition to forced convection
 - Transition to natural convection
- MOTEL experimental program:
 - Helical HX heat transfer, pressure drop
 - Cross flow in the core

KIT Approach for SMR Core Analysis: Multiphysics

KIT Approach: Multiphysics

- Industry-like approach: Nodal diffusion / 1D system TH
 - PARCS / TRACE or PARCS/ RELAP5 [6]

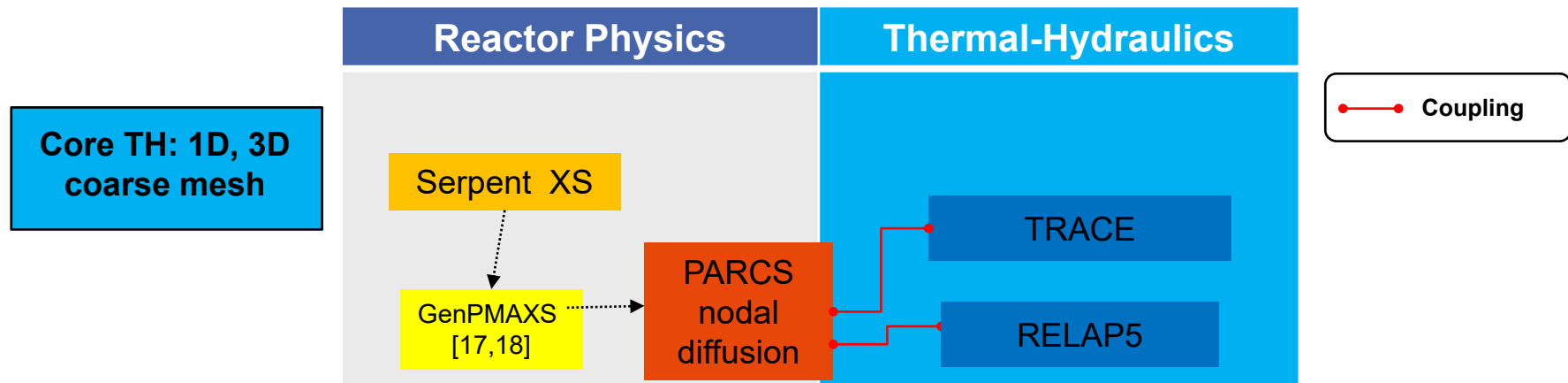
- Improved core thermal hydraulics:
 - Subchannel codes: Subchanflow (in-house) [7,8]
 - Porous-media 3D TH: Twoporflow (in-house) [9]

- Improved neutronics:
 - Simplified transport solvers at pin level:
 - PARCS-SP3

 - Monte Carlo codes:
 - Static simulations: Serpent2 [10]/Subchanflow/ICoCo [11]
 - Dynamic simulations: internal coupled Serpent2/Subchanflow [13,14,15]

 - Deterministic transport solvers e.g. PARAFISH (in-house) [12]

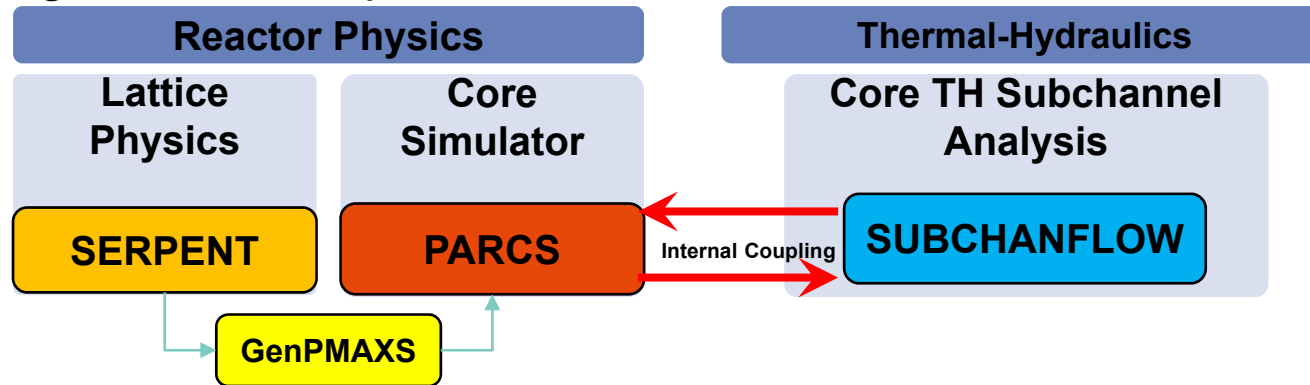
KIT Core Analysis Tools: Internal Coupling



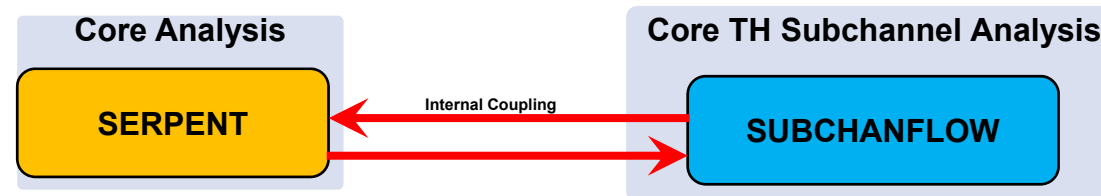
No direct prediction of **local** safety parameters

Internal coupling: PARCS/SCF & SERPENT2/SCF (2019)

- Core design and core optimization tools:



- 3D core reference solutions:



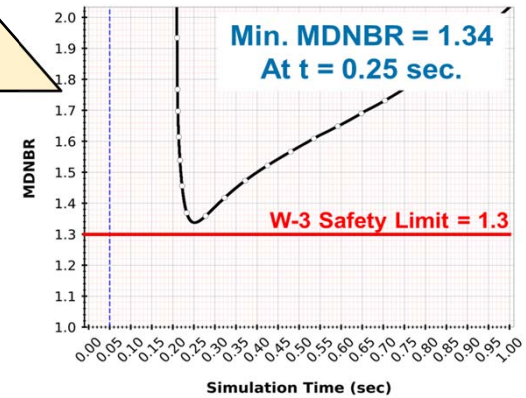
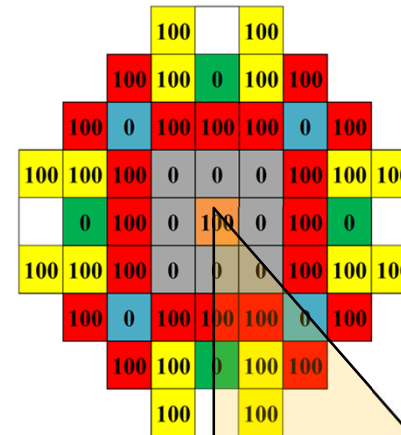
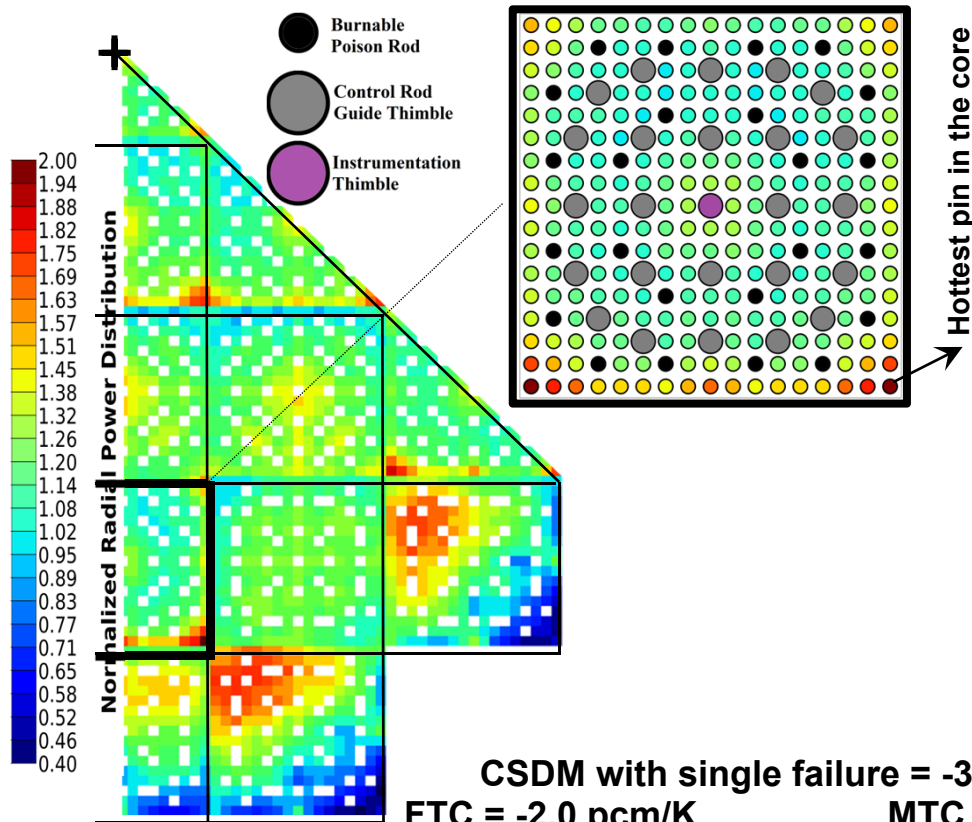
Visualization Tool



KSMR:

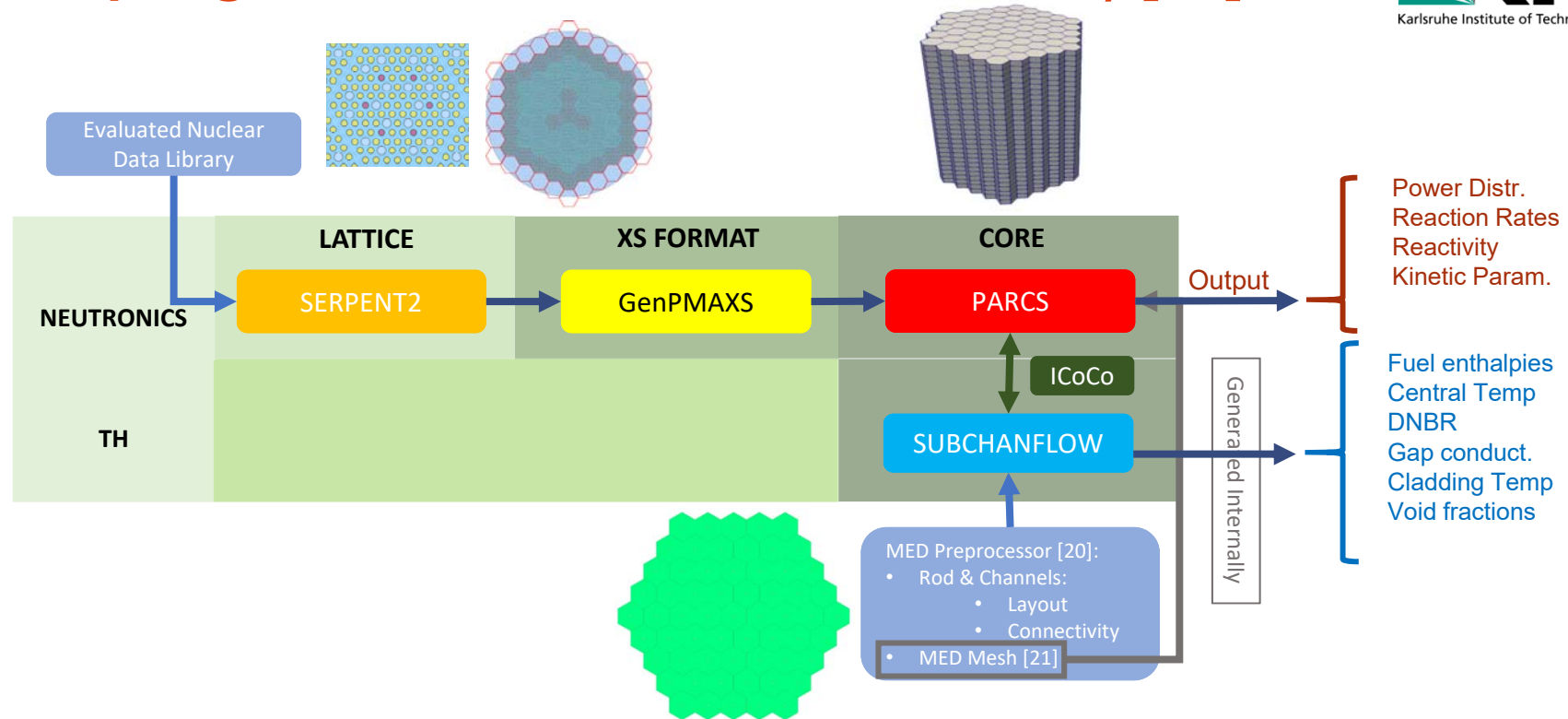
rel. radial pin Power Distribution at HFP [3,4,5]

Ref. [4,5]

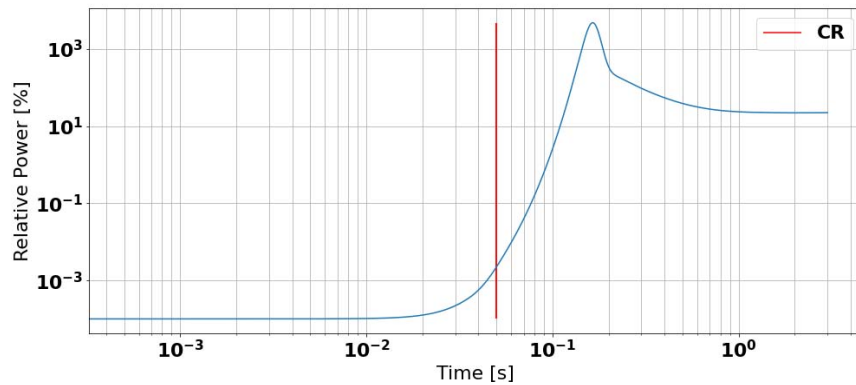


REA: MDNBR

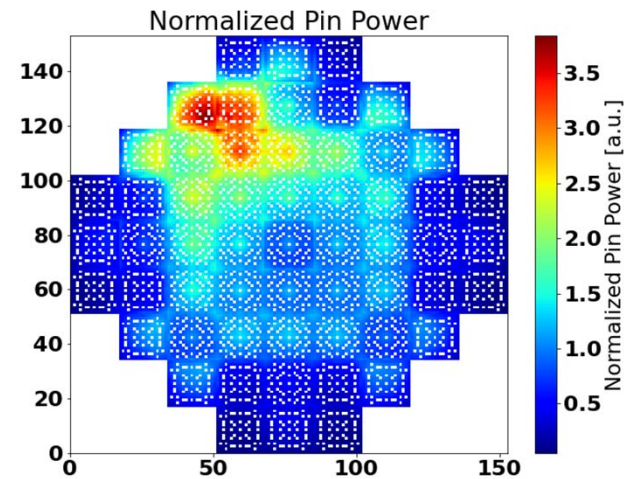
ICoCo-based Coupling of PARCS/ Subchanflow (2021) [18]



KSMR REA: ICoCo PARCS/SCF Analysis [20]

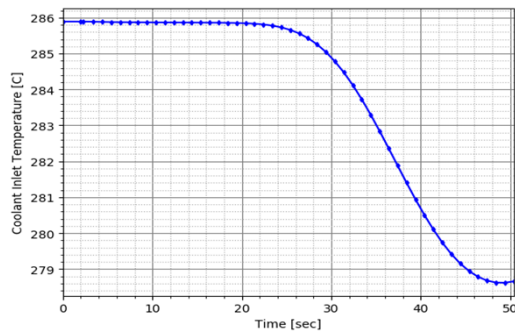


KSMR Rod Ejection Accident with Pin Power Reconstruction at power peak

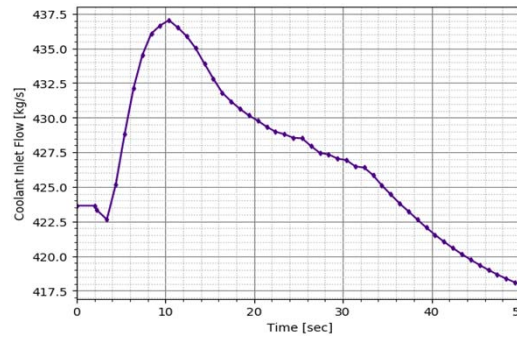


KSMR REA: Relative power [%] at peak power

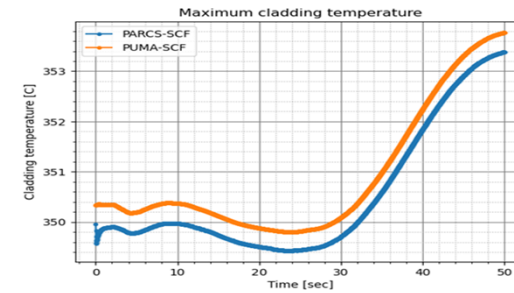
PARCS/SCF/ICoCo: Analysis of CAREM: Overcooling Transient [20]



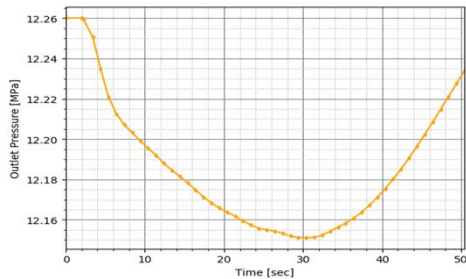
Coolant temperature decrease



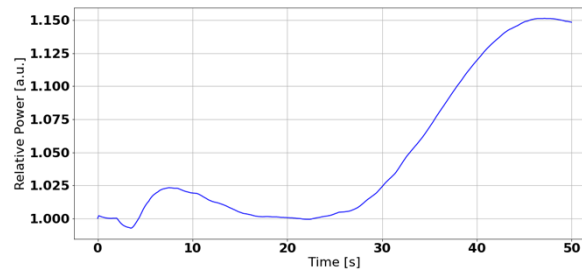
Mass flow rate change



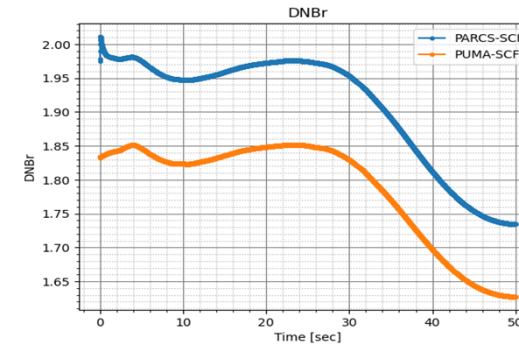
CAREM Overcooling transient: Key-parameters



Outlet pressure evolution



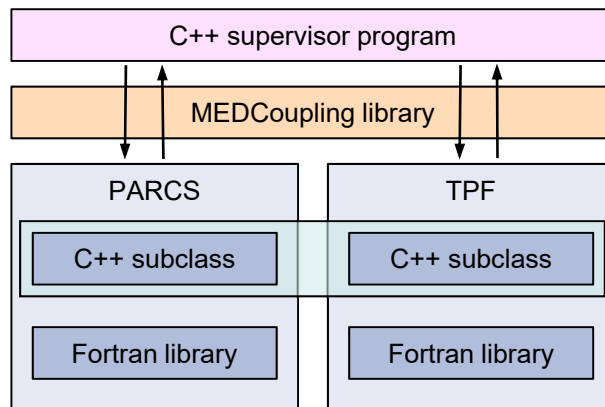
Normalized Power [a.u.]



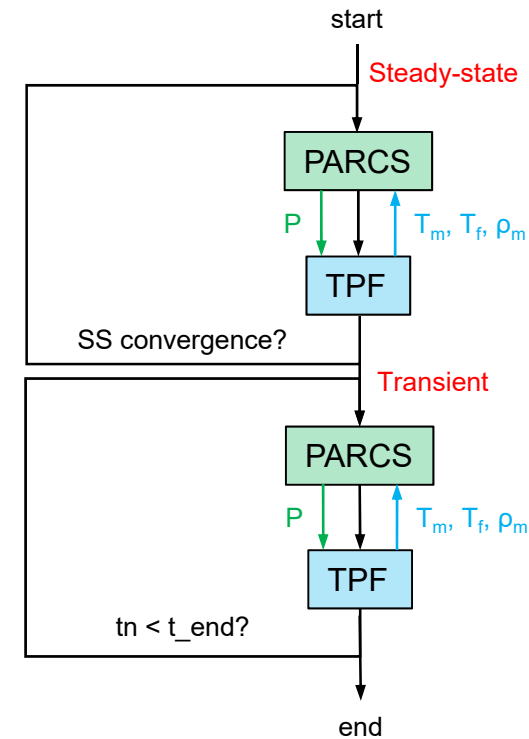
Ref. [20]

ICoCo-based coupling of PARCS/TwoPorflow (2022)

- External coupling.
- Serial execution.
- Domain overlapping.
- Fields mapping via MEDCoupling library.
- Explicit iterative scheme.
- Node-wise feedback.



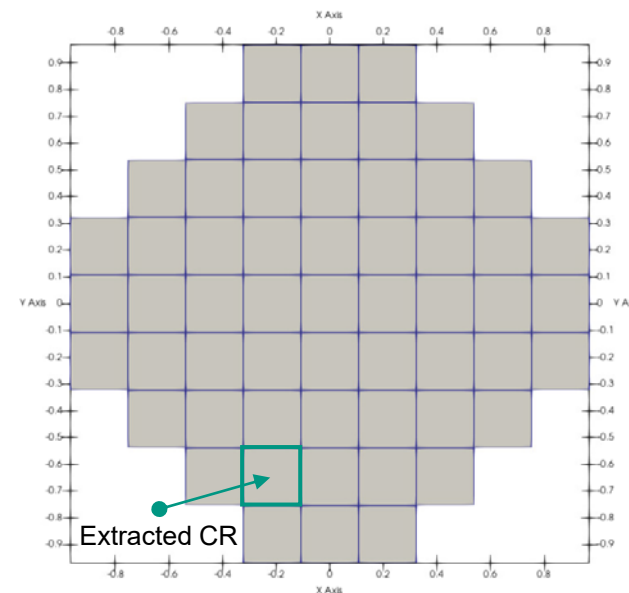
Ref. [9]



KSMR: REA Analysis at HZP

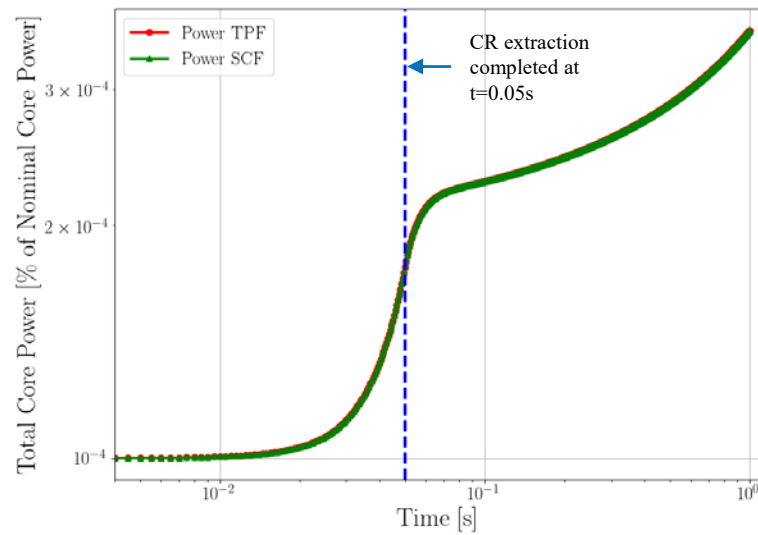
- Half highest CR worth extraction (0.725 \$) at the hot zero power (HZP) condition.

Parameter	Value
Initial core power	1.0E-4 %
Highest CR worth	1.45 \$
Ejection duration	0.05 s
End of transient simulation	1.0 s

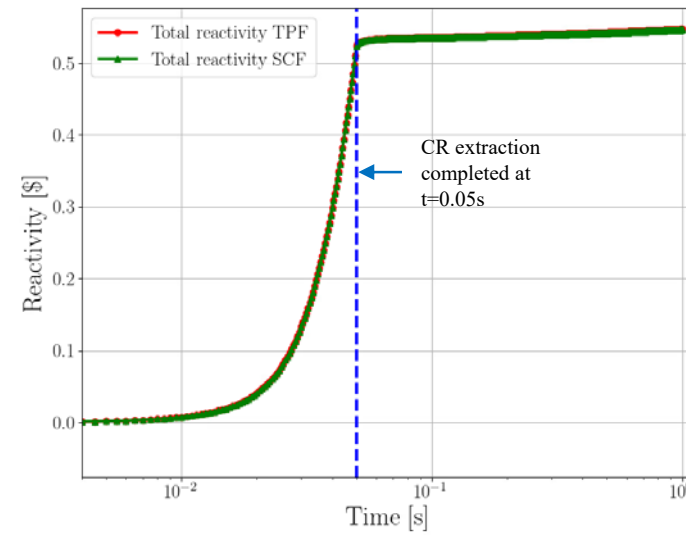


Ref. [9]

SMART REA Analysis: PARCS/SCF vs. PARCS/TFP



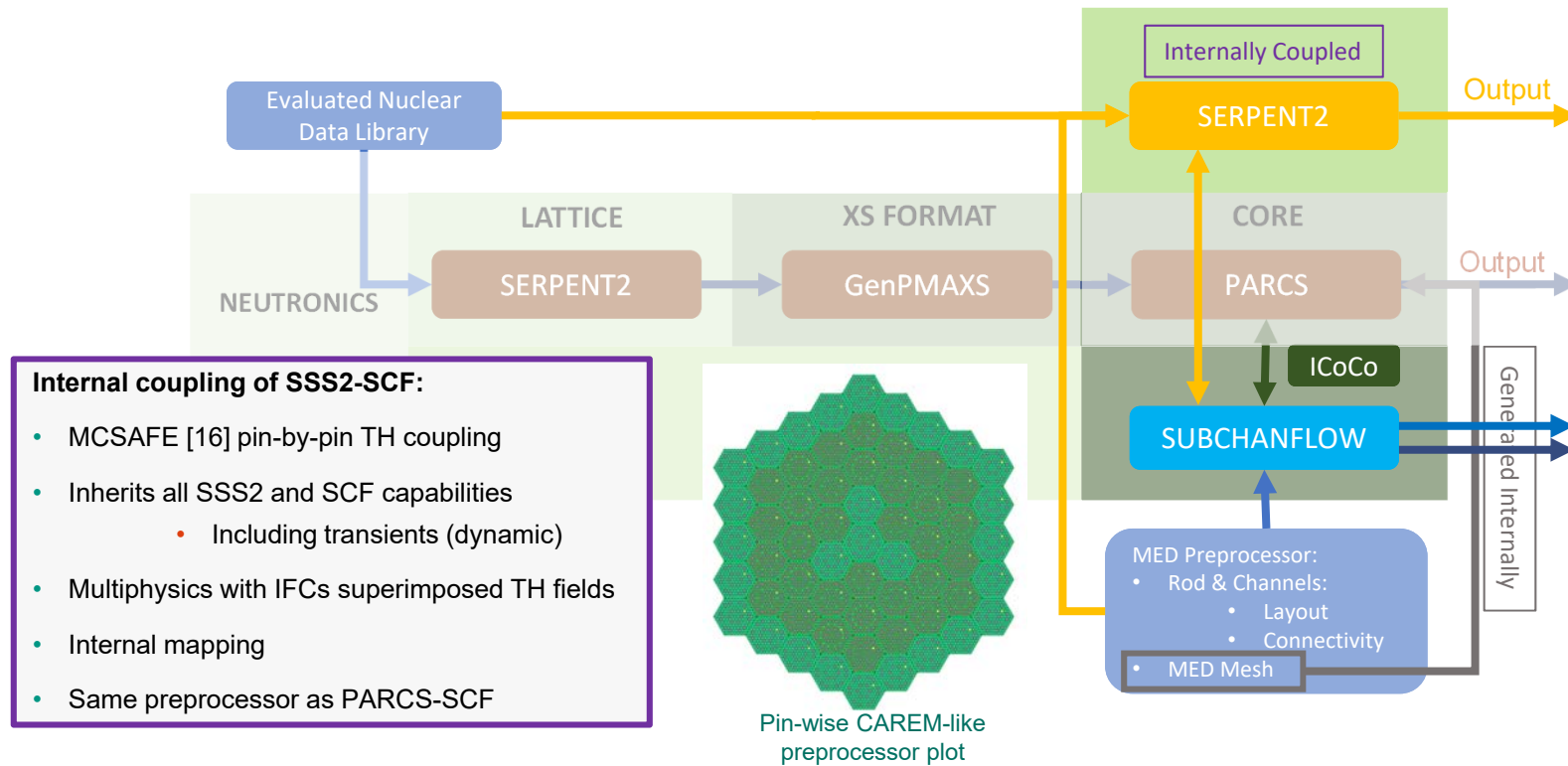
PARCS Total Core Power



PARCS Total Reactivity

Ref. [9]

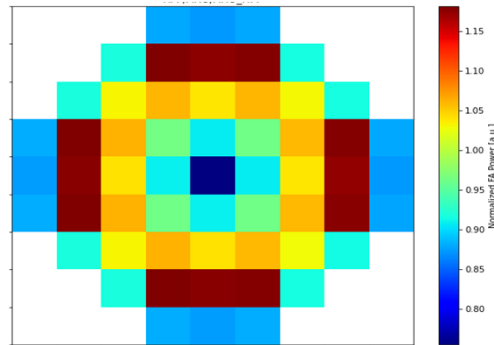
High Fidelity MC-based Multiphysics: SSS2-SCF



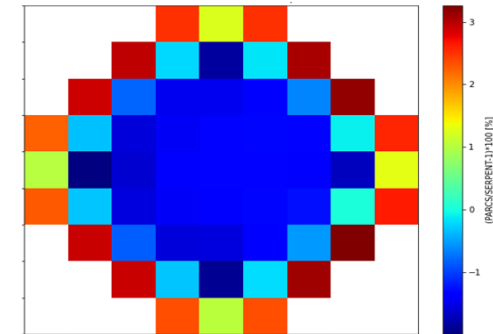
Ref. [20, 13,14,23,24]

KSMR: Comparison of PARCS/SCF and SSS2/SCF Solutions [22]

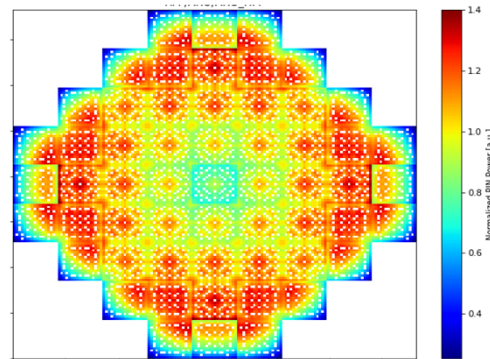
SSS2: axially integrated FA normalized radial power for HFP ARO



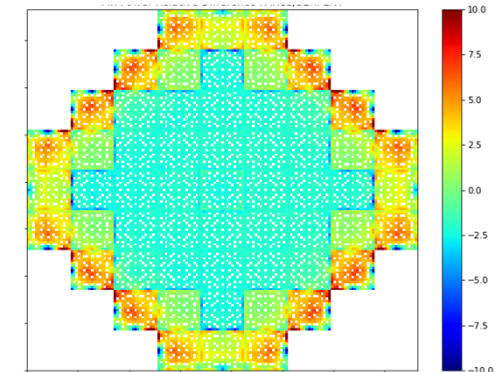
PARCS-SSS2: axially integrated FA normalized radial power relative difference for HFP ARO



SSS2: axially integrated pin normalized radial power for HFP ARO



PARCS-SSS2: axially integrated pin normalized radial power relative difference for HFP ARO. Values are truncated in the interval [-10; 10] %

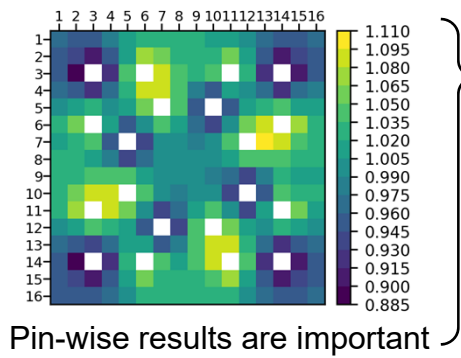


Ref. [20]

Outlook

Next steps: Simplified transport solver PARCS-SP3 pin-by-pin (1/2)

- Motivation: Pin-wise Simulation, XS Optimization, TH Feedback



In KIT, we would like to do:

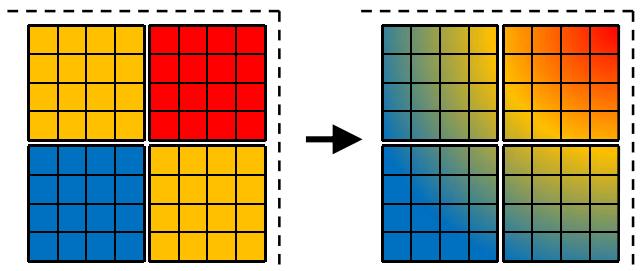
- Pin-by-pin simulation in core-scale with "ASSY_TYPE".
- Enable pin-wise XS optimization and TH feedback.

PARCS V331 can not do this

Function extension is required

PARCS V331 has two methods for pin-wise results:

- Nodal + Pin power reconstruction



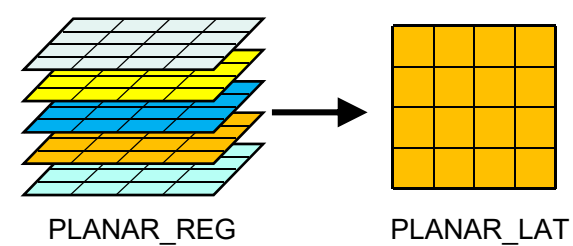
Advantage:

- Fast running.

Limitation:

- No Pin-wise TH coupling.
- No Pin-wise XS optimization.

- FMFD (Fine Mesh Finite Difference)



Advantage:

- SP3 Pin-wise simulation.

Limitation:

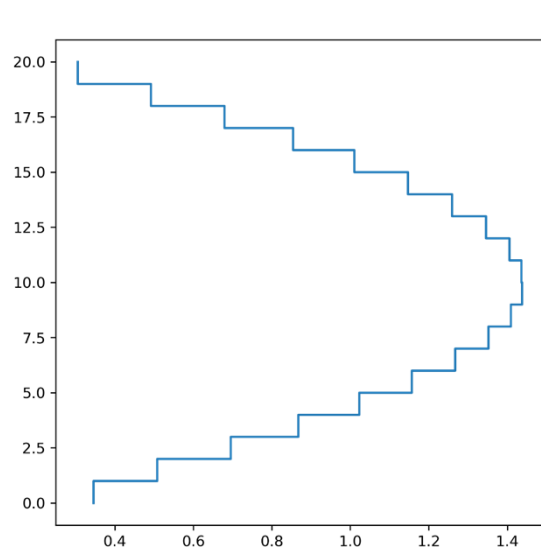
- Use "PLANAR_REG", no "ASSY_TYPE".

The discussion in this slide only concern Cartesian geometry

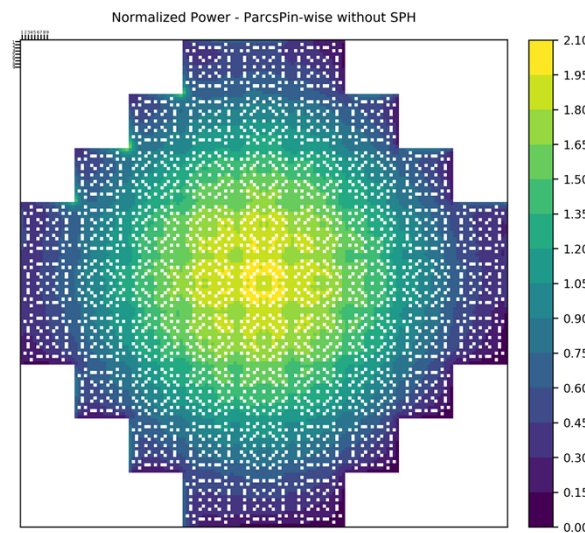
Ref. [22]

Next steps: Simplified transport solver PARCS-SP3 pin-by-pin (2/2)

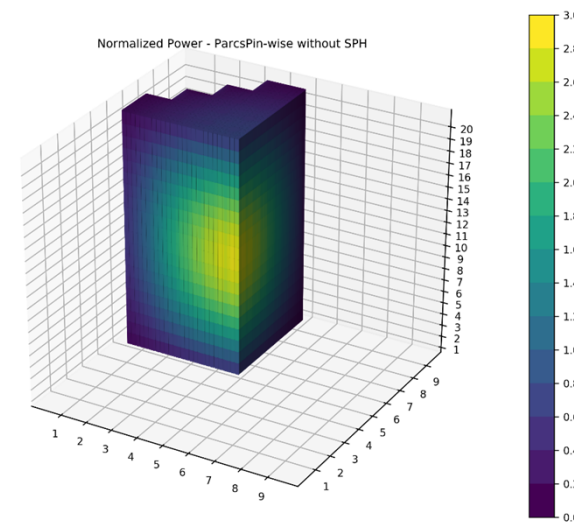
- First Results: KSMR core steady state



Axial power
distribution



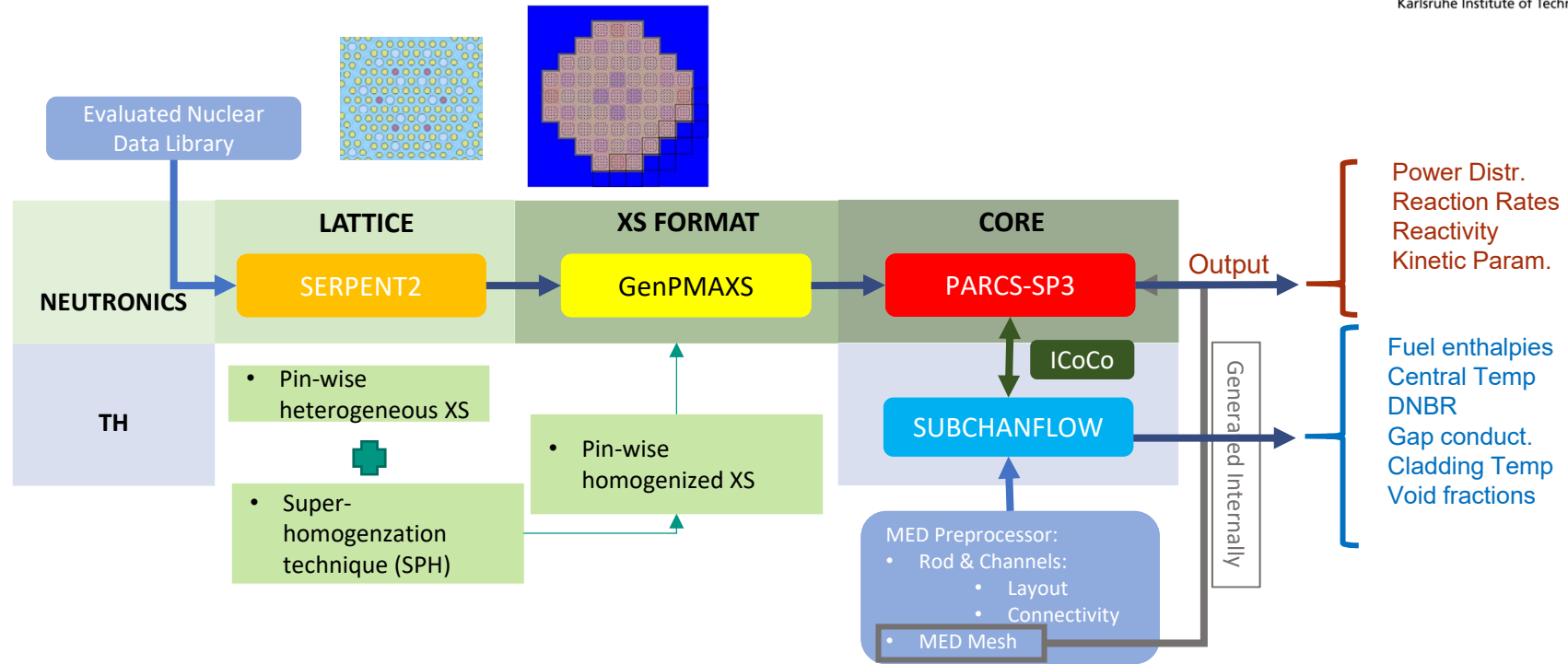
Radial power
distribution



Spatial power
distribution

Ref. [22]

Next steps: Simplified transport solver PARCS-SP3 pin-by-pin (2/2)



Pin /subchannel coupling based on same ICoCo-Interface FA-based Coupling

Next steps: High fidelity Transport Solver

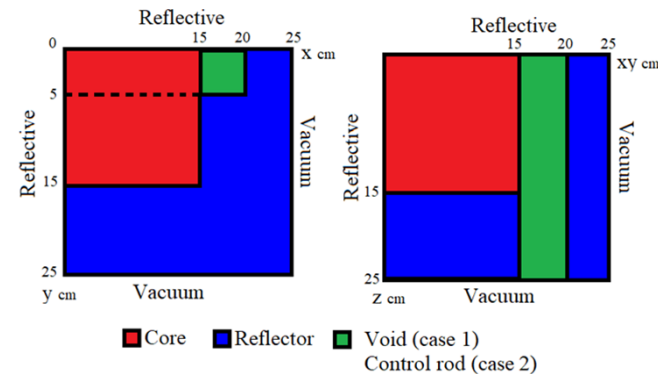
■ PARAFISH code development

- Finite element spherical harmonic neutron transport solver
- Written in C++
- Domain decomposition

■ At present: steady state solver

■ Next steps:

- Time-dependent solver
- Coupling with TH e.g. SCF
- Pin-by-pin analysis of SMR-core transients
- Code-to-code comparisons
 - E.g. SSS2/SCF



Code	Case 1	Case 2	CR-worth
Monte-Carlo	0.97780	0.96240	1.64×10^{-02}
Parafish (P ₁₁)	0.97686 Error= 96 pcm	0.96249 Error= 9 pcm	1.52×10^{-02} Error= 7.3 %

Small PWR: LWR based on the Kyoto University Critical Assembly (KUCA)

Ref. [12]

References

1. High-Performance Advanced Methods and Experimental Investigations for the Safety Evaluation of Generic Small Modular Reactors– McSAFER. Horizon 2020. , Proposal number 945063, (2020).
2. VALTAVIRTA, V., FARD, A., FRIDMAN, E., LESTANI, H., and MERCATALI, L., McSAFER D3.1: Specifications for the reactivity transients scenarios in the four SMR cores, (2021).
3. Y. Alzaben, V.H. Sanchez-Espinoza, R. Stieglitz; Analysis of a steam line break accident of a generic SMART-plant with a boron-free core using the coupled code TRACE/PARCS. NED 350 (2019)33-42. <https://doi.org/10.1016/j.nucengdes.2019.05.002>
4. Y. Alzaben, V.H. Sanchez-Espinoza, R. Stieglitz; Analysis of a control rod ejection accident in a boron-free small modular reactor with coupled neutronics/thermal-hydraulics code. ANE 134 (2019)114-124. <https://doi.org/10.1016/j.anucene.2019.04.017>
5. Y. Alzaben, V. Sanchez-Espinoza and Stieglitz, "Core neutronics and safety characteristics of a boron-free core for Small Modular Reactors," ANE 132 (2019)70-81, <https://doi.org/10.1016/j.anucene.2019.04.017>
6. DOWNAR, T. et al., PARCS. NRC - v3.3.1 Release. Volume I: Input Manual, Division of Risk Assessment and Special Projects Office of Nuclear Regulatory Research U. S. Nuclear Regulatory Commission Washington, DC 20555-0001 TRACE V5.0 USER'S MANUAL. Volume 1: Input Specification
7. IMKE, U., User manual for SUBCHANFLOW 3.7.1, (2021).
8. IMKE, U. et al., 2012. Validation of the subchannel code SUBCHANFLOW using the NUPEC PWR tests (PSBT). Sci. Technol. Nucl. Install. 2012, 12. <https://doi.org/10.1155/2012/465059>. URL <http://downloads.hindawi.com/journals/stni/2012/465059.pdf>
9. Alejandro Campos Muñoz; Coupling of PARCS with TPF. KIT Internal report May 2022.
10. LEPPÄNEN, J., PUSA, M., VIITANEN, T., VALTAVIRTA, V., and KALTAISENAHO, T., The Serpent Monte Carlo code: Status, development and applications in 2013, Ann. Nucl. Energy 82 (2015) 142.
11. The ICoCo API, <https://docs.salome-platform.org/7/dev/MEDCoupling/icoco.html>
12. J. Duran-Gonzalez, V. H. Sanchez-Espinoza, L. Mercatali, A. Gomez-Torres and E. d. Valle-Gallegos, "Verification of the parallel transport codes Parafish and AZTRAN with the TAKEDA Benchmarks," Energies, vol. 15, pp. 8346-8368, 2022.
13. FERRARO, D. et al., 2020. Serpent/SUBCHANFLOW pin-by-pin coupled transient calculations for a PWR minicore. Annals of Nuclear Energy 137, 107090. <https://doi.org/10.1016/j.anucene.2019.107090>.
14. FERRARO D., GARCIA, M., VALTAVIRTA V., IMKE, U., TUOMINEN, R., LEPPÄNEN J., SANCHEZ-ESPINOZA, V.; Serpent/SUBCHANFLOW pin-by-pin coupled transient calculations for the SPERT-III hot full power tests. ANE 42(2020)107387.
15. MERCATALI L. et al., 2017. McSafe projects Highlights – Available in NUGENIA project portfolio, available at URL: <https://www.ne.ncsu.edu/event/workshopinternational-multi-physics-validation/>, Presentation at International MultiPhysics Validation Workshop North Carolina State University (June 2017)..
16. WARD, A. . X. Y. . D. T., GenPMAXS – V6.2. Code for Generating the PARCS Cross Section Interface File PMAXS, Michigan, (2016).
17. WARD, A. . X. Y. . D. T., GenPMAXS – V6.3 Release. Code for Generating the PARCS Cross Section Interface File PMAXS, (2020).
18. GARCIA, M. et al., 2019. Advanced Modelling Capabilities for Pin-level Subchannel Analysis of PWR and VVER Reactors. In: 18th International Topical Meeting on Nuclear Reactor Thermal Hydraulics (NURETH-18).
19. CEADEN EFD R&D, OPEN CASCADE. (Version: 5.1.6). Salome Platform Documentation. MEDMEM library,. <https://docs.salome-platform.org/5/med/user/medmem.html>
20. FRIDMAN, E. et al., McSAFER D3.4: State-of-the-art solutions for the transient scenarios in the four SMR cores, (2022)
21. Victor Hugo Sanchez-Espinoza, 1, Stephan Gabriel, Heikki Suikkanen, Joonas Telkkä, Ville Valtavirta, Marek Bencik, Sören Kliem, Cesar Qeral, Arthime Farda, Florian Abéguié, Paul Smith, Paul Van Uffelen, Luca Ammirabile, Marcus Seidl, Christophe Schneidesch, Dmitry Grishchenko, Hector Lestani; The H2020 McSAFER project: Main goals, technical work program, and status. Energies 2021, 14(19), 6348; <https://doi.org/10.3390/en14196348>
22. K. Zhang, L. Mercatali, J. Blanco, .V. Sanchez-Espinoza; Optimized Cross-Section (XS) Group Generation for PARCS from SERPENT Statics based on the SuPer-Homogenisation (SPH) Technique. D3.5 McSAFER H2020 Project. 2021
23. Manuel Garcia, Radim Vocka, Riku Tuominen, Andre Gommlich, Jaakko Leppänen, Ville Valtavirta, Uwe Imke, Diego Ferraro, Paul Van Uffelen, Lukas Milisdörfer, Victor Sanchez-Espinoza; Validation of Serpent-SUBCHANFLOW-TRANSURANUS pin-by-pin burnup calculations using experimental data from the Temelin II VVER-1000 reactor. Nuclear Engineering and Technology. Nuclear Engineering and Technology Volume 53, Issue 10, October 2021, Pages 3133-3150. <https://doi.org/10.1016/j.net.2021.04.023>
24. Manuel Garcia, Yuri Bilodid, Joaquin Basualdo Perello, Riku Tuominen, Andre Gommlich, Jaakko Leppänen, Ville Valtavirta, Uwe Imke, Diego Ferraro, Paul Van Uffelen, Marcus Seidl, Victor Sanchez-Espinoza; Validation of Serpent-SUBCHANFLOW-TRANSURANUS pin-by-pin burnup calculations using experimental data from a Pre-Konvoi PWR reactor. NED 379 (2021) 111173. <https://doi.org/10.1016/j.nucengdes.2021.111173>
25. Diego Ferraro, Ville Valtavirta, Manuel Garcia, Uwe Imke, Riku Tuominen, Jaakko Leppänen, Victor Sanchez-Espinoza; OECD/NRC PWR MOX/UO2 core transient benchmark pin-by-pin solutions using Serpent/SUBCHANFLOW. ANE 147 (Nov 2020) 107745. <https://doi.org/10.1016/j.anucene.2020.107745>
26. Manuel Garcia, Manuel Garcia, Diego Ferraro, Ville Valtavirta, Riku Tuominen, [Uwe Imke, Jaakko Leppänen](https://doi.org/10.1016/j.anucene.2019.106955), Victor Sanchez-Espinoza; Serpent2-SUBCHANFLOW pin-by-pin modelling capabilities for VVER geometries. ANE 135 (2020 January)106995 <https://doi.org/10.1016/j.anucene.2019.106955>
27. Diego Ferraro, Manuel Garcia, Ville Valtavirta, Uwe Imke, Riku Tuominen, Jaakko Leppänen, Victor Sanchez-Espinoza; Serpent/SUBCHANFLOW pin-by-pin coupled transient calculations for a PWR minicore; ANE 137 (March 2020) 107090. <https://doi.org/10.1016/j.anucene.2019.107090>.
28. Nicholas R. Brown, Andrew Worrall, "Fuel Cycle Performance of Thermal Spectrum Small Modular Reactors", ICAPP 2016, San Francisco, CA, April 17-20, 2016

