

Neutronic analyses for EU DEMO upper limiter

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Introduction

- Limiters play a crucial role in the operation of DEMO
- Upper limiter integration was done in several steps (Talk: M. L. Richiusa et al - P6A4, Thursday afternoon)
- · Simplistic models used in initial analyses
 - Radial dependence of the nuclear loads ("1D") to investigate the general behavior and determine suitable cooling channel density
 - Effect of different shielding mixtures on the deposited nuclear loads
- · Effect of different shielding mixtures on the neutron/gamma shielding performance
- Detailed models based on initial results used in more detailed analyses
 - · More detailed modelling, but still simplistic, i.e. cooling channels not explicitly modelled Detailed 3D maps of the nuclear loads - to study local behavior
 - Design of the cooling solution was optimized based on these results (Poster: A. Froio et al - PS3-3, Thursday)
- This work is part of ISFNT paper: M. L. Richiusa et al, The Integrated Engineering Design Concept of the Upper Limiter within the EU-DEMO LIMITER System

Conclusions

- Simple model for initial investigations, detailed model to determine local behavior
- · No problems found with shielding performance
- Detailed 3D maps of nuclear heating. DPA, and He production
- · Integral and peak values of nuclear loads informed future design directions

1. Tolls used

- · SuperMC [1] for CAD to MCNP geometry conversion
- · MCNP5 v1.60 [2] and JEFF 3.3 [3] nuclear data for analyses

2. Simple limiter model

- Initial simulations carried out using simple model · 3 plasma facing layers
- · Shielding block (homogenized water-Eurofer mix)
- · Shield block (SB) configurations homogenized Box design – Eurofer box with water filling
 - · Plate design Eurofer plates cooled by water channels
- Analysis of radial dependence of nuclear heating
- HCPB and WCLL tritium breeding blanket (TBB)

Tab 1: Material composition of analyzed cases for the simple limiter design



3. Analyses using simple limiter model

- Total nuclear heating of each layer
- Radial dependence of nuclear heating
- · For different SB configurations and TBB concepts Minor differences in deposited energy for HCPB Significantly different heating radial profile in WCLL









Fig 1: Simple model of the upper limite

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4. Detailed model of upper limiter

- · More realistic geometry
- 3 plasma facing layers + attachment layer Shielding block still homogenized - based
- on plate design

Tab 3: Material composition of layers for the detailed limiter design

Layer	material composition
PFC-1 (20 mm)	Tungsten (100%)
PFC-2 – layer with	Cu (11.70 vol%), CuCrZr (14.81 vol%),
cooling pipes	water (26.33 vol%, at 0.935 g/cm3),
(17 mm)	tungsten (47.17 vol%)
PFC-3 (3 mm)	Tungsten (100%)
PFC attachment	Eurofer (21 vol%), void (79 vol%)
layer (23 mm)	
Shielding block	Eurofer (92.85 vol%), water (5.28 vol% at
	0.7 g/cm ³), alumina (1.87 vol%)

Fig 3: Detailed model of the upper limiter

5. Analyses using a detailed model

- Nuclear loads of interest
 - · Nuclear heating in limiter (total per component and mesh based)
 - · DPA, He production in limiter
 - Peak nuclear heating in TFC: 6 W/m3 (limit 50 W/m3) good shielding properties
 - DPA in vacuum vessel (VV): 0.2 DPA / 6 FPY (limit 2.75 DPA) VV is far from the limiter

Laver

PFC-

PEC-2

PFC-3 PFC attachment layer





Tab 4: Total nuclear heating per component

Nuclear heating

2.35 MW

1 46 MW

0.376 MW

0.171 MW



Fig 6: Neutron induced damage in Eurofer [DPA/FPY]

6. Further work

- · Upper limiter as a reference for other limiters
- · Initial work already done on
 - · Outboard midplane limiter (OML)
 - · Outboard lower limiter (OLL)

References

[1] Y. Wu, FDS Team, CAD-based interface programs for fusion neutron transport simulation, Fusion Engineering and Design 84 (2009), 1987-1992. [2] F. Brown et al., Verification of MCNP5-1.60, LA-UR-10-05611, Los Alamo: National Laboratory (2010).
 [3] JEFF 3.3: https://www.oecd-nea.org/dbdata/jeff/jeff33/

: Simple neutronics model used for initial OML and OLL analyses. Fig

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