



ISFNT 15

INTERNATIONAL SYMPOSIUM ON FUSION NUCLEAR TECHNOLOGY

10-15
SEPT 2023

AUDITORIO
ALFREDO KRAUS
LAS PALMAS DE
GRAN CANARIA,
SPAIN



Alternative water-cooled BB concepts for the EU-DEMO: Overview on studies and perspectives

15th International Symposium on Fusion Nuclear Technology

Gran Canaria, 12th September 2023

F.A. Hernández¹, P. Arena², G. Bongiovi³, I. Catanzaro³, C.A. Cerviño⁴, S. D'Amico⁵, A. Del Nevo², P.A. Di Maio³, G. Federici⁵, C. Garnier⁶, S. Giambone³, J.C. Marugán⁴, I. Moscato⁵, I. Palermo⁷, P. Pereslavytsev¹, A. Quartararo³, F. Roca Ugorri⁷, G.A. Spagnuolo⁵, A. Tassone⁸, G. Zhou¹

¹ Karlsruhe Institute of Technology, Germany

² ENEA C.R. Brasimone, Italy

³ University of Palermo, Italy

⁴ Empresarios Agrupados, Spain

⁵ EUROfusion PMU, Germany

⁶ CEA Cadarache, France

⁷ CIEMAT, Spain

⁸ University of Rome – La Sapienza, Rome, Italy

Breeding Blanket Project in  EUROfusion



This work has been carried out within the framework of the EUROfusion Consortium, funded by the European Union via the Euratom Research and Training Programme (Grant Agreement No 101052200 — EUROfusion). Views and opinions expressed are however those of the author(s) only and do not necessarily reflect those of the European Union or the European Commission. Neither the European Union nor the European Commission can be held responsible for them.



- 1. Reference BB concepts & motivation to explore variants**
2. Exploring a WCLL variant: the WCLL „double bundle“
3. Exploring a WCLL-HCPB hybrid variant: WLCB
4. Summary and Outlook



2. Reference BB concepts & motivation for variants

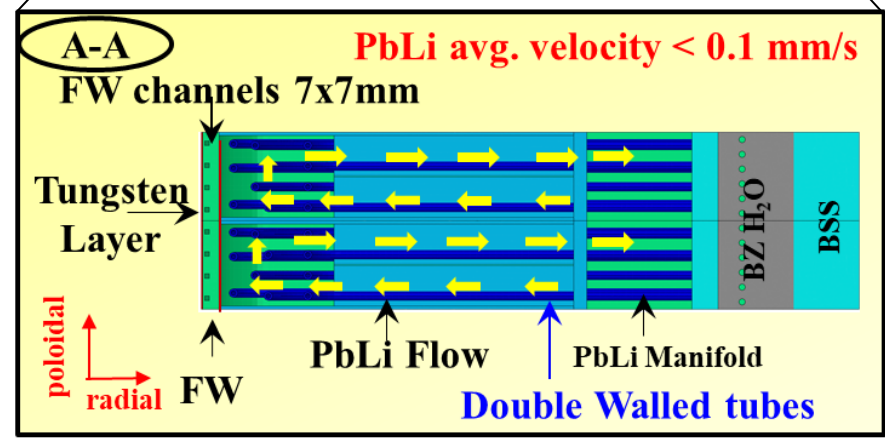
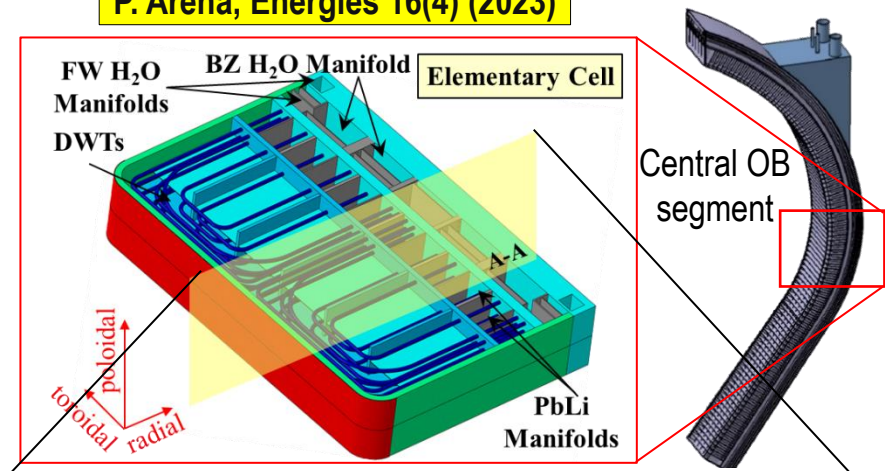


P. Arena, Energies 16(4) (2023)

- Current WCLL baseline reference variant
 - DEMO: 16 sectors, 3 OBS + 2 IBS per sector, SMS segments
 - PWR water cooling (295-328°C, 15.5MPa), 2 loops (FW, BZ)
 - PbLi as n-multiplier, T-breeder and carrier
 - Unit cells cooled by radial Double Wall Tubes
 - PbLi radial flow in BZ, poloidal flow in manifold
 - Structural steel: EUROFER97, W-armor 2mm

■ Identified risks as of end PCD phase:

Risk ID	Risk
1	Low reliability of BB system
2	Low efficiency of PbLi draining
3	FW based on thin EUROFER + W-armor
5	Low T breeding performance
6	Large amount of transmutation helium in PbLi
10	Large T permeation to coolant
12	WCLL operating with EUROFER temp. irradiated <400 °C (DBTT shift)
13	Pressure transient uncertainties due to PbLi-water interaction
22	Diffusion of Li into anti-permeation barriers and production of T+He there



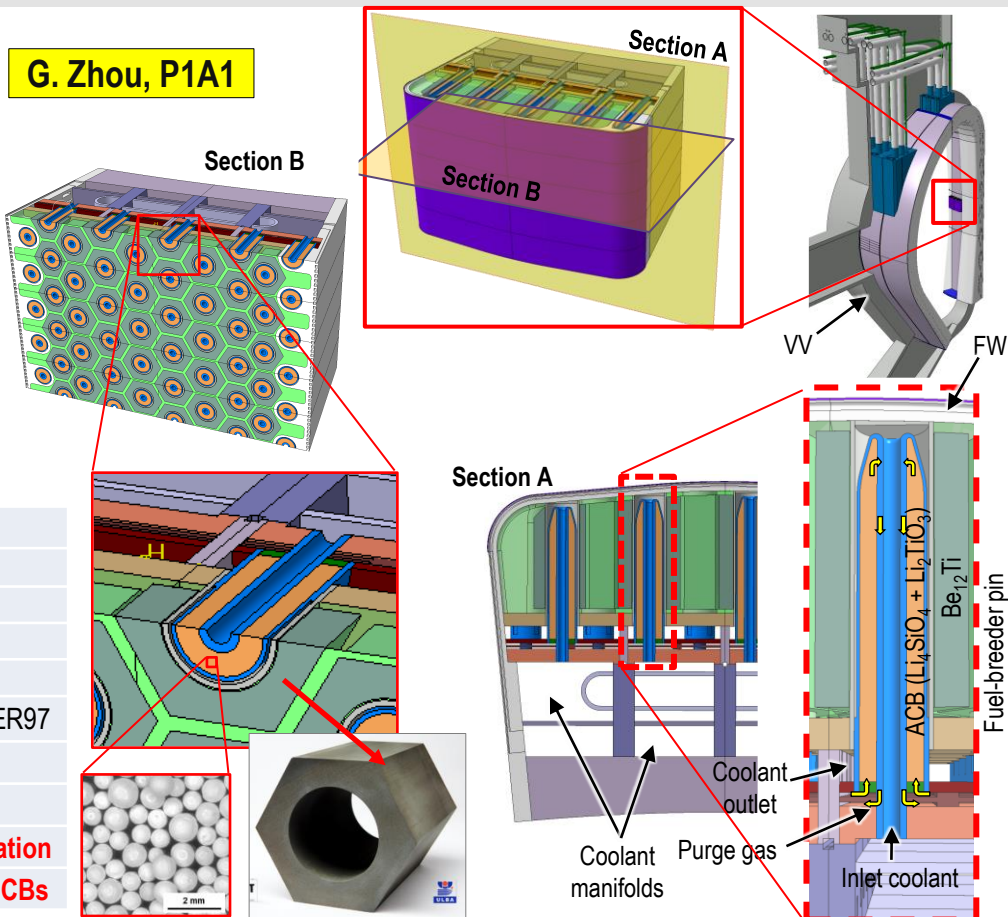
2. Reference BB concepts & motivation for variants



- Current HCPB baseline reference variant
 - DEMO: 16 sectors, 3 OBS + 2 IBS per sector, SMS
 - He cooling (300-520°C, 8 MPa), 1 loop (FW + BZ)
 - Be₁₂Ti blocks as n-multiplier, Li₄SiO₄ + Li₂TiO₃ as T-ceramic breeder (ACB), He purge gas and T carrier
 - Unit : hexagonal fuel-breeder pin arrangement
 - Structural steel: EUROFER97, W-armor 2mm

- Identified risks as of end pre-CD phase:

HCPB	Risk ID	Risk
	1	Low reliability of BB system
	2	Limited heat flux removal capability of the blanket FW
	3	Loss of structural integrity Be ₁₂ Ti blocks
	4&9	Low TRL industrial production Be ₁₂ Ti blocks and CBs
	7	Reduction of structural integrity of BB due to DBTT shift of EUROFER97
	11	Large T permeation to coolant
	14	Low BB shielding capability
	18&19	Unknown behavior of Be ₁₂ Ti and ceramic breeder under irradiation
	22	Deterioration of mech. properties of EUROFER97 in contact w/ CBs





1. Reference BB concepts & motivation to explore variants
- 2. Exploring a WCLL variant: the WCLL „double bundle“**
3. Exploring a WCLL-HCPB hybrid variant: WLCB
4. Summary and Outlook



3. Exploring a WCLL variant: WCLL “double bundle”

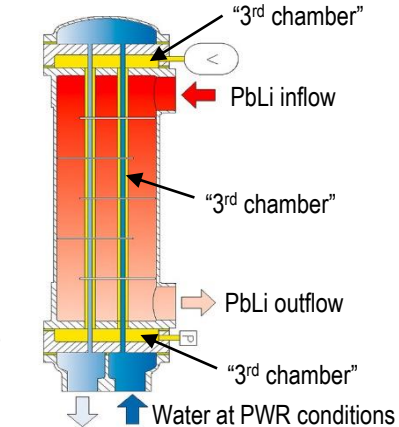
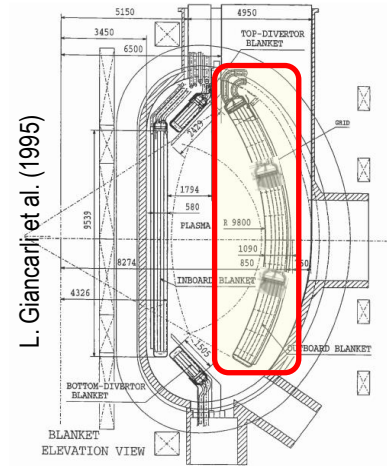


Motivation:

WCLL	ID	Risk	Addressed by
	1	Low reliability of BB system	(1)
	2	Low efficiency of PbLi draining	(4)
	3	FW based on thin EUROFER + W-armor	limiters
	5	Low tritium breeding performance	(3)
	6	Large amount of transmutation He in PbLi	
	10	Large T permeation to coolant	(2)(6)
	12	WCLL operating with EUROFER temp. irradiated <400 °C (DBTT shift)	
	13	Pressure transient uncertainties due to PbLi-water interaction:	(5)
	22	Diffusion of Li into anti-permeation barriers and production of He there	(2)(6) may avoid barriers

Poloidal water tube distribution:

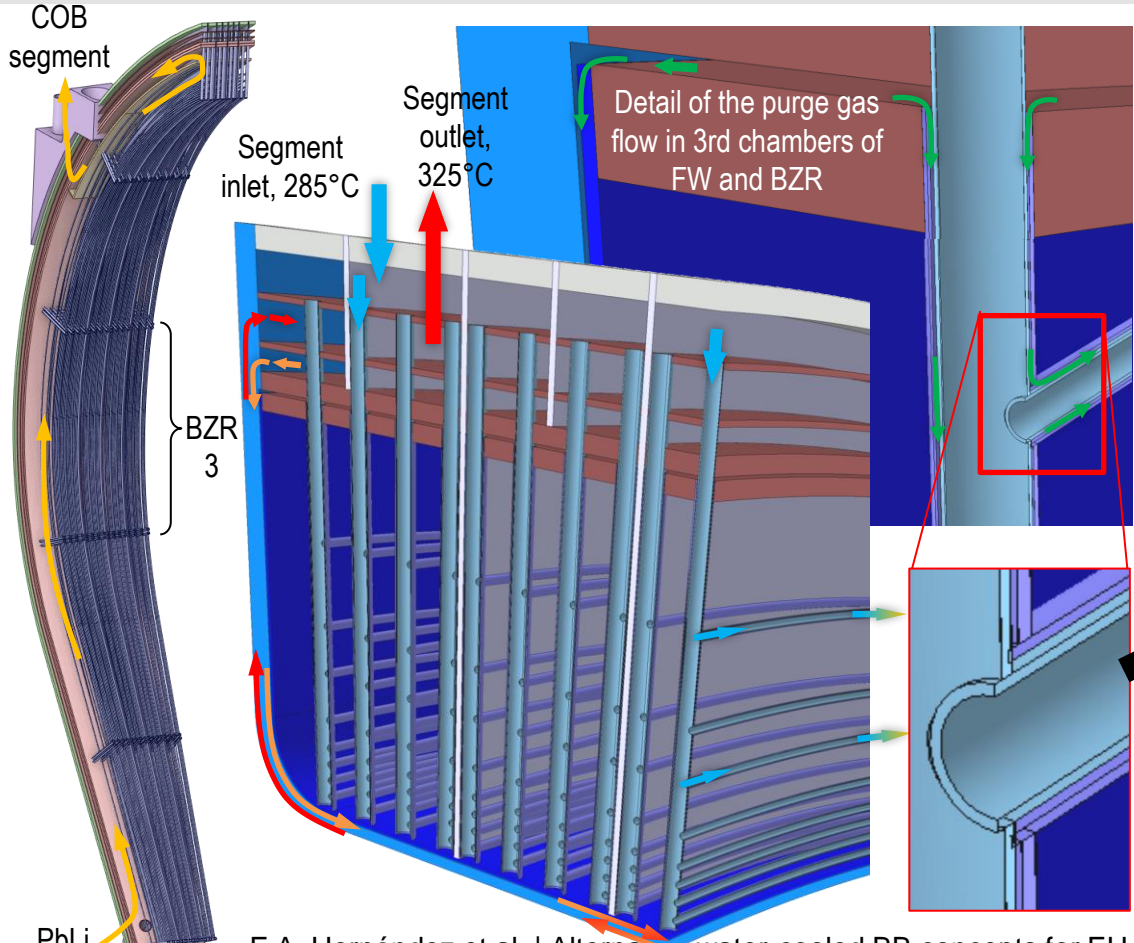
- Poloidal tubes:
 - (1) Less tubes, less welds, ↑ reliability
 - (2) Less tubes, less surface, ↓ T-permeation
 - (3) Less tubes, less water, more PbLi, ↑ TBR
 - (4) Easier draining and less He accum. risks
- BB similar to HX/SG => ↑ TRL/RoX
- Segments split in several poloidal regions
 - Limit heat flux per tube
 - Allows systems integration (H/CD, limiters...)
 - w/o splitting segments



„Double bundle“ of simple tubes

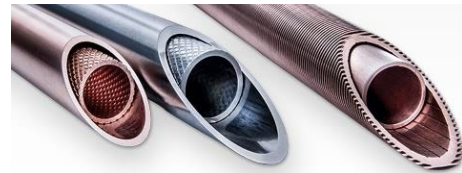
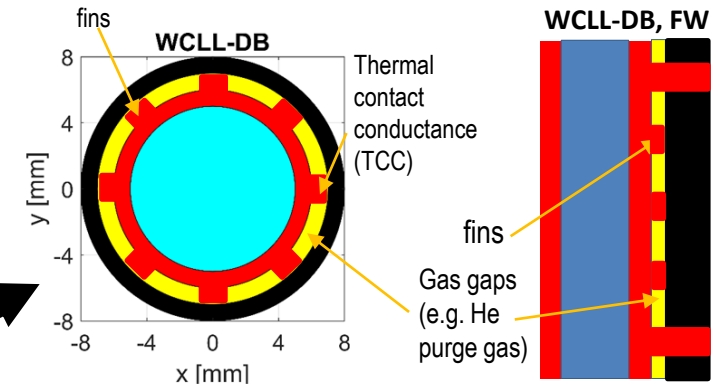
- 3-chamber idea of S&T HX (K.-H. Funke)
 - (5) Intermediate chamber between PbLi and water to avoid contact in case of internal LOCA
 - (6) 3rd chamber filled with He gas: used to remove permeated T before it reaches water

3. WCLL-DB: Conceptual design



Design description:

- PWR cooling (285-325°C, @15.5 MPa)
- BZ and FW in series ($T_{in,FW} \approx 315^\circ\text{C}$)
- 5 BZRs, poloidal PbLi flow
- „3rd chamber“ between the double bundle, filled with a He purge gas



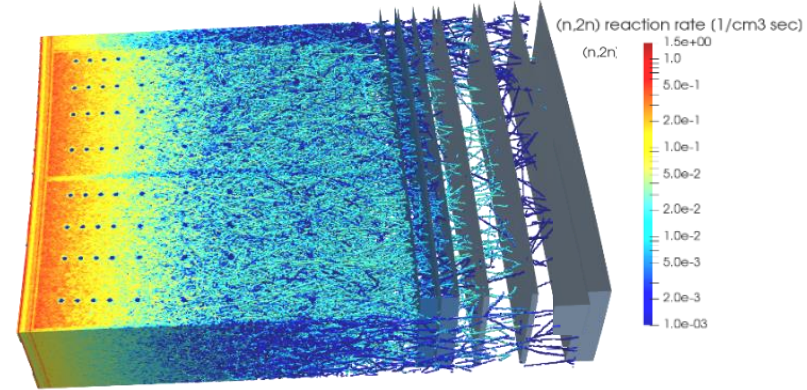
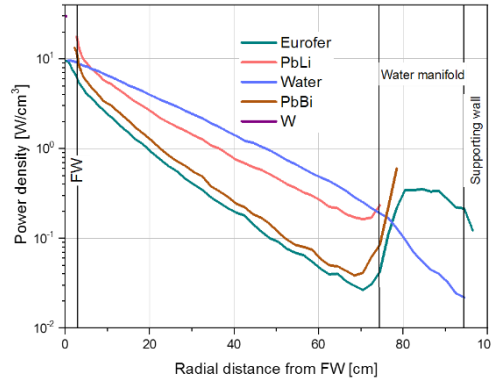
E.g. Wieland Safety Tubes:
<https://www.wieland-thermalsolutions.com>

3. WCLL-DB: Neutronics

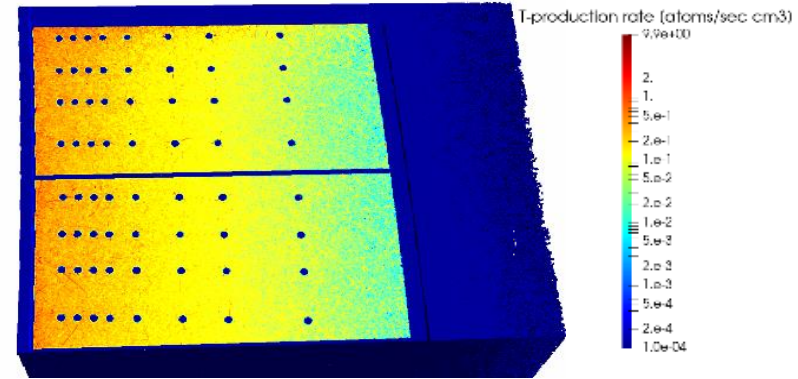
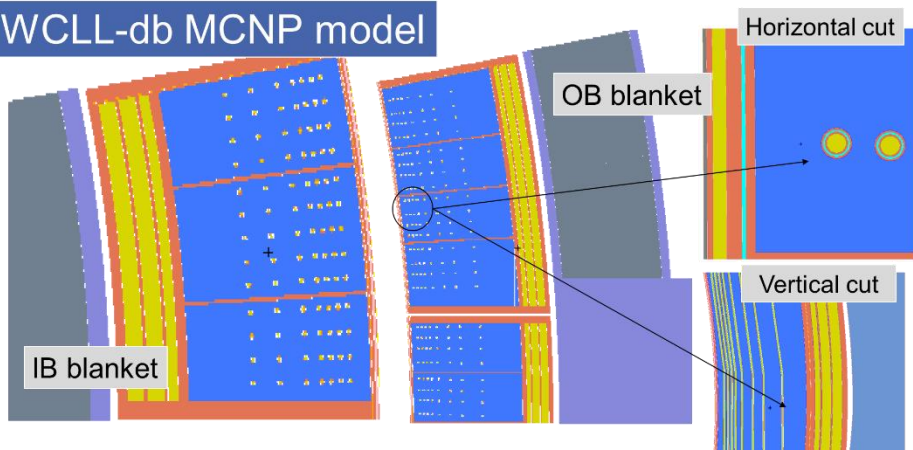


Summary 2021: Neutronics

- MCNP-6.2, JEFF-3.3 library
- 3D WCLL DEMO sector (11.25°)
- Fully heterogeneous model
- **TBR = 1.16** (ref. 10 FW ch/BU)
- **TBR = 1.17** (6 FW ch/BU)
- Water manif. large => possibility to enlarge BZ ($\approx +0.01-0.02$)



WCLL-db MCNP model

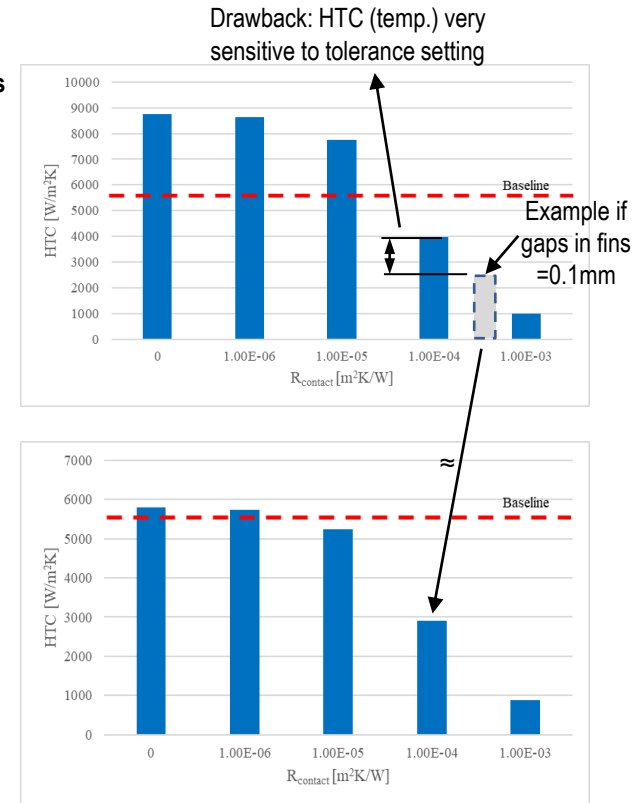
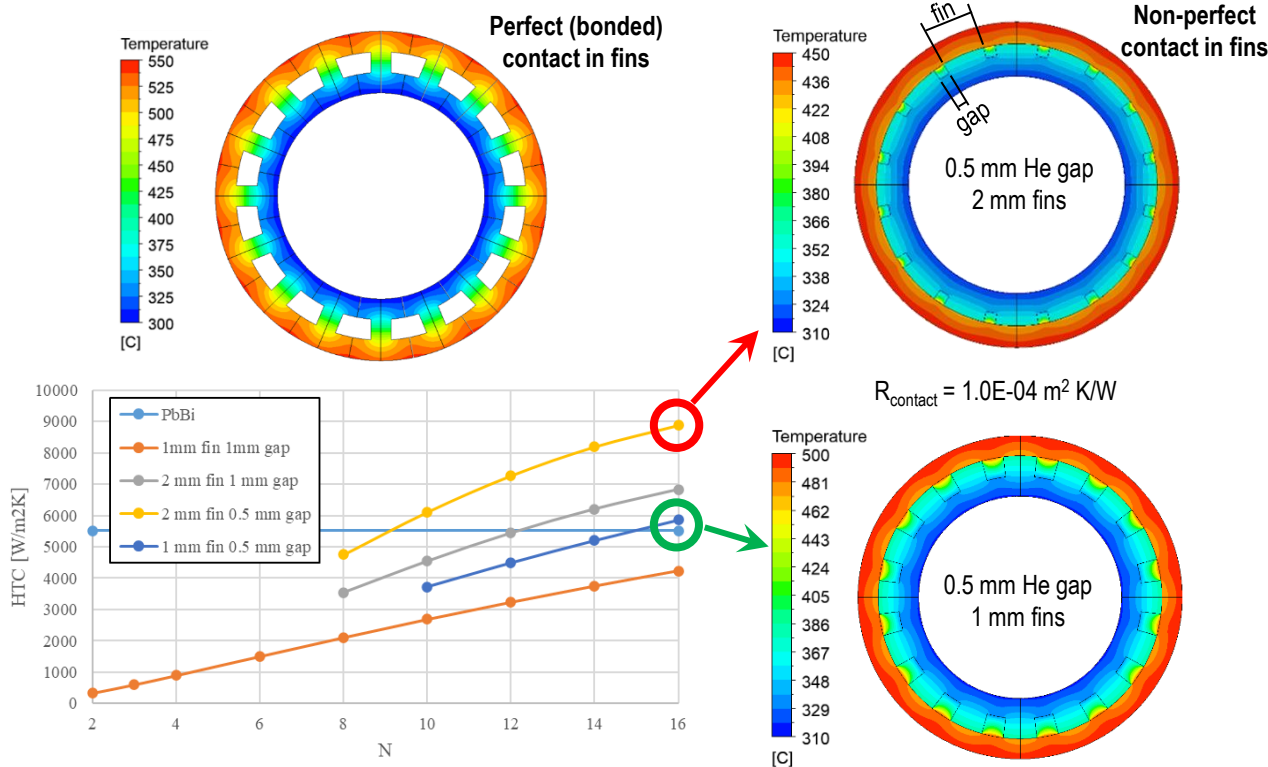


3. WCLL-DB: Thermal-hydraulics



- Parametric study changing #fins (2 - 16), their thickness (1; 2 mm) and gap height (0.5; 1 mm)
- Each HTC_{global} (gap + fins) compared to 2021 baseline (PbBi interlayer)

P.A Di Maio, PS2-45

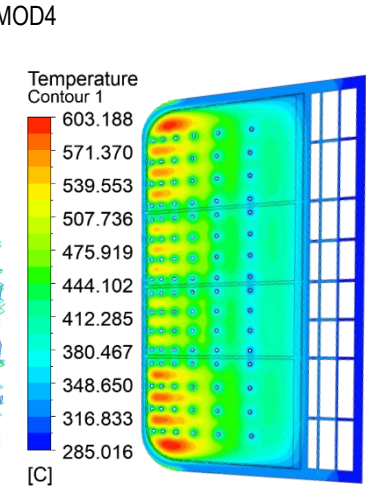
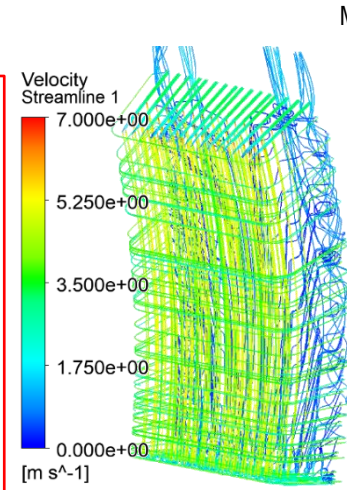
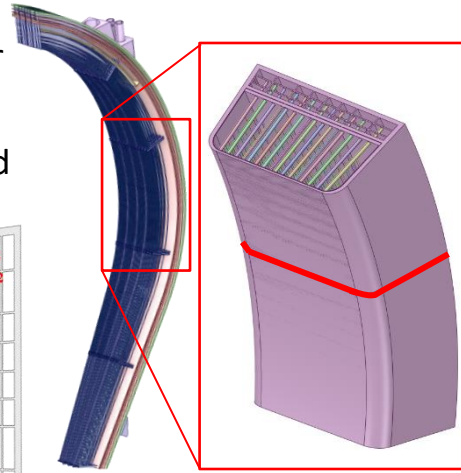
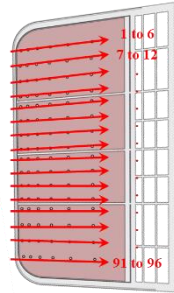
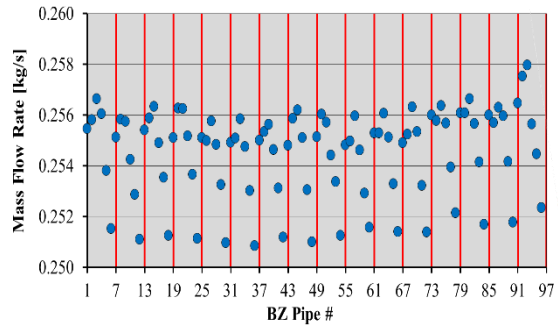


3. WCLL-DB: Thermal-hydraulics-mechanics



Summary: Thermohydraulics

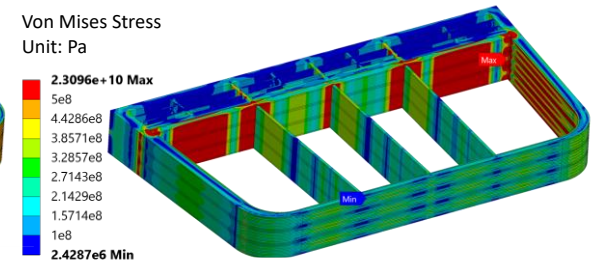
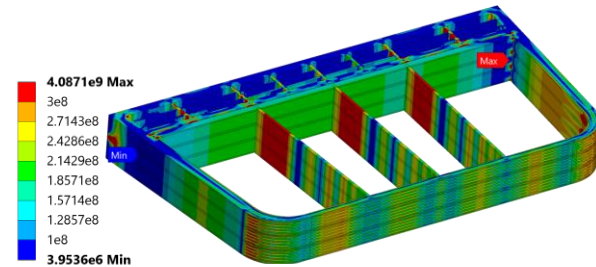
- BZR3: $\Delta p_{FW} = 0.555$ bar, $\Delta p_{BZ} = 0.693$ bar
- Mass flow distribution homogeneous
- Heat transfer through fins demonstrated



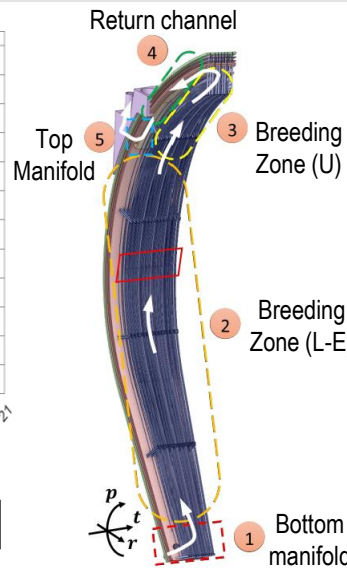
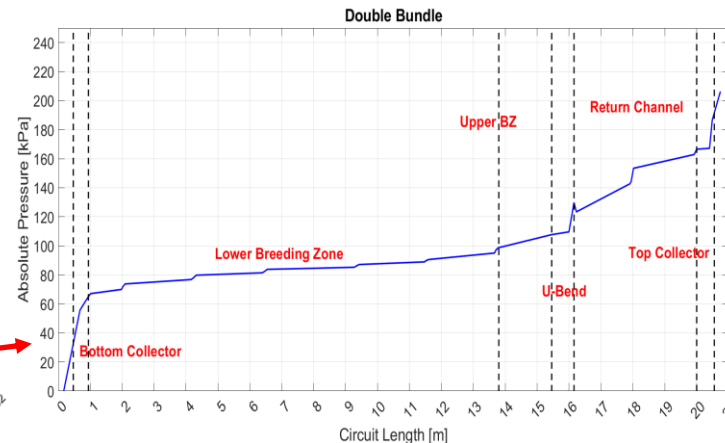
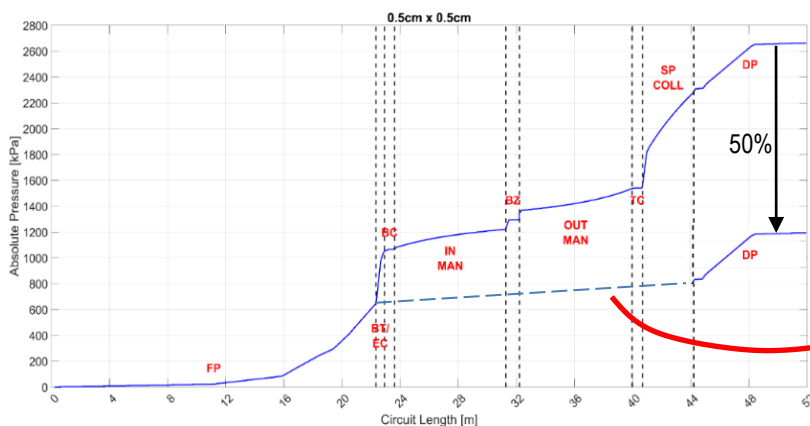
P.A Di Maio, PS2-45

Summary: Thermomechanics

- Parametric assessment for NO and OP
- Problematic regions seem easily solved by local reinforcement of structures



3. WCLL-DB: MHD analyses



Methodology

- $\Delta p_{MHD} = \Delta p_{2D} + \Delta p_{3D}$ semi-emp. correlation + RELAP5 benchmark
- PbLi flow: bottom + BZ (Low-Eq, upper, return) + top manifold

Assumptions

- Only toroidal field, no EM coupling between channels
- Hydrodynamic friction and concentrated losses neglected, $T_0 = 600$ K
- No effect of Δp due to streamwise obstacles (tubes)

Outcome

- $\Delta p_{WCLL-db} \approx 0.1 \Delta p_{WCLL}$, R5 and correl. good agreement ($\epsilon \approx 3\%$)

A. Tassone, PS2-30

	WCLL-db	WCLL-db RELAP5*	WCLL [1]
Total Δp [kPa]	151.5	156.0	1512.0

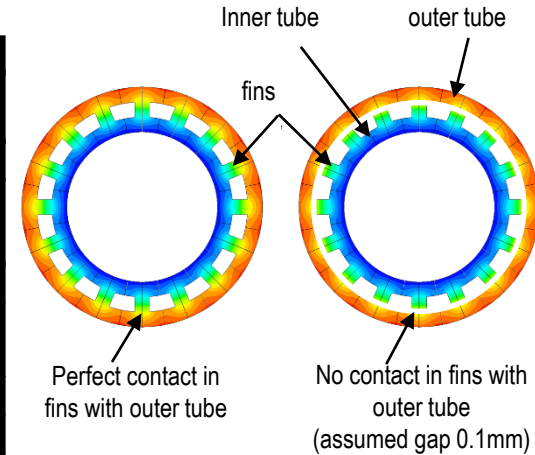
*Reference side channel for WCLL-db [1] 2022 WCLL Design Team Meeting

3. WCLL-DB: Tritium transport analyses



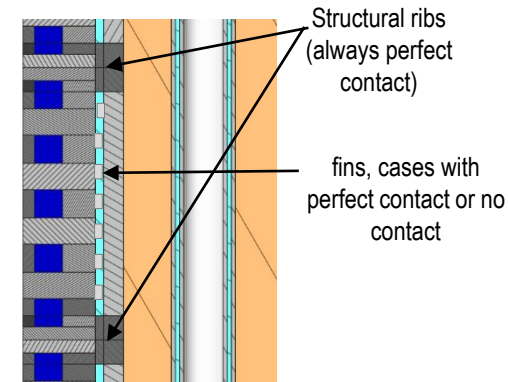
Simplified T-transport analyses

Permeation rates to water circuits (g/d)	WCLL reference (PRF = 1)	WCLL-DB					
		Stagnant purge gas	Flowing purge gas (\dot{m} as in HCPB pin)				
		495 °C	495 °C	330 °C	495 °C	330 °C	
		Perfect contact in fins	Perfect contact in fins	Perfect contact in fins	No contact in fins	No contact in fins	No contact in fins, 10x \dot{m}
Water tubes	44.18	27.668	7.071	2.194	8.2E-02	2.5E-02	2.4E-03
Feeding manifolds	--	4.878	9.0E-01	2.7E-01	9.3E-03	2.8E-03	2.7E-04
First wall	2.38	4.330	5.2E-01	1.4E-01	5.3E-01	1.3E-02	1.3E-02
Back wall	--	5.8E-01	2.2E-02	1.1E-02	3.5E-03	1E-02	1E-02
Total	46.56	37.458	8.515	2.614	6.3E-01	1.7E-01	1.5E-01



Conclusions:

- Stagnant PG, modest reduction ($\approx 1.24x$)
- Temperature has a significant effect ($\approx 3x$)
- Flowing PG and perfect contact in fins, significant reduction ($\approx 5x - 18x$)
- Flowing PG and imperfect contact in fins, massive reduction ($\approx 74x - 319x$)
 - Potential to eliminate barriers, but HTC very sensitive to fins tolerances
- Dominant perm. path \Rightarrow FW/back wall structural ribs, impact of PG \dot{m} limited



3. WCLL-DB: Reliability/FMEA analysis



Summary: Scenarios with highest yearly failure rate (FR)

- Multiplicities: WCLL-ref > WCLL-DB > WLCB. Yearly FR has to be read together with its consequence

Failure and element	WCLL-Ref			WCLL-DB			Consequence
	FR min [1/y]	FR max [1/y]	Multip.	FR min [1/y]	FR max [1/y]	Multip.	
Leak/rupture F/T pipe PbLi	4.91E-02	4.09E-01	416	6.56E-02	5.47E-01	416	In-VV leak
Leak/rupture of poloidal welds between LiPb-BP and FW	2.57E-01	3.69E+00	72576	N/A			In-box LOCA
Leak of the LiPb-BP double welds	2.10E+00	2.33E+00	354816	N/A			In-box LOCA
Leak/rupture of pol. and tor. welds in the LiPb outlet manif.	2.34E+00	3.36E+01	661248	3.03E-04	4.34E-03	64	Bypass
Leak/rupture of weld of water pipes with water feed in/out feeder manifold halves	sw	N/A		3.62E-01	5.19E+00	76544	In-tube LOCA
	dw	N/A		3.62E-02	5.19E-01		
Leak/rupture weld of purge gas pipes with purge gas in/out feeder manifold halves	sw	N/A		3.62E-01	5.19E+00	76544	In-tube leak PbLi
	dw	N/A		3.62E-02	5.19E-01		
Leak/rupture of purge gas feeder manifold	N/A			2.78E-01	7.41E+00	11008	In-tube leak PbLi
Leak/rupture of purge gas chamber in FW	N/A			1.10E+00	2.94E+01	10000	In-tube leak PbLi
Leak/rupture of purge gas poloidal tubes	ST	N/A		3.62E+00	9.66E+01	38272	In-tube leak PbLi
	DWT	N/A		2.11E-03	3.62E-02		
Leak/rupture weld connection of the manif to the manif. from next breeder zone region	sw	N/A		2.72E-02	3.25E-01	4800	In-box LOCA
	dw	N/A		2.72E-03	3.25E-02		
Loss of structural integrity of the purge gas chamber	N/A			N/A			In-VV leak p.g.
Leak/rupture of structural weld of water manif. to the water CPs	N/A			N/A			In-box LOCA

Legend
sw = single welds
dw = double welds (with DWT)
ST = Simple Tube
DWT = Double Wall Tube

- ← DWT + dw recommended
- ← DWT + dw recommended
- ← C-shaped DWT recommended
- ← FW purge gas chambers HIPed
- ← DWT recommended

Recommendations & outlook:

- Implement C-shaped DWT with double welds (instead of ST+ feeders) to decrease FR of these elements
- Consequence of in-tube leaks still to be understood (further operation possible?), positive effect of leak monitoring



1. Reference BB concepts & motivation to explore variants
2. Exploring a WCLL variant: the WCLL „double bundle“
- 3. Exploring a WCLL-HCPB hybrid variant: WLCB**
4. Summary and Outlook

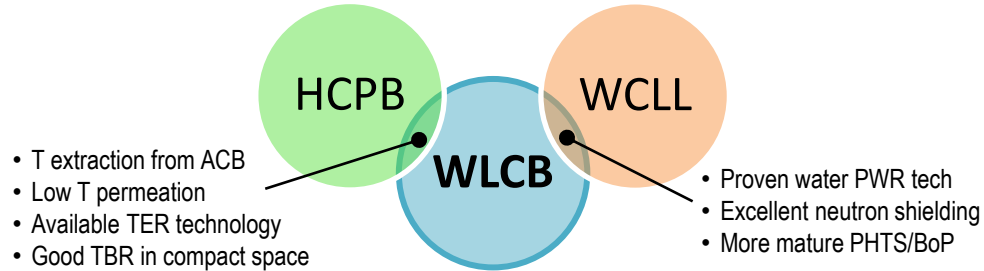


5. WLCB: Initial conceptual design



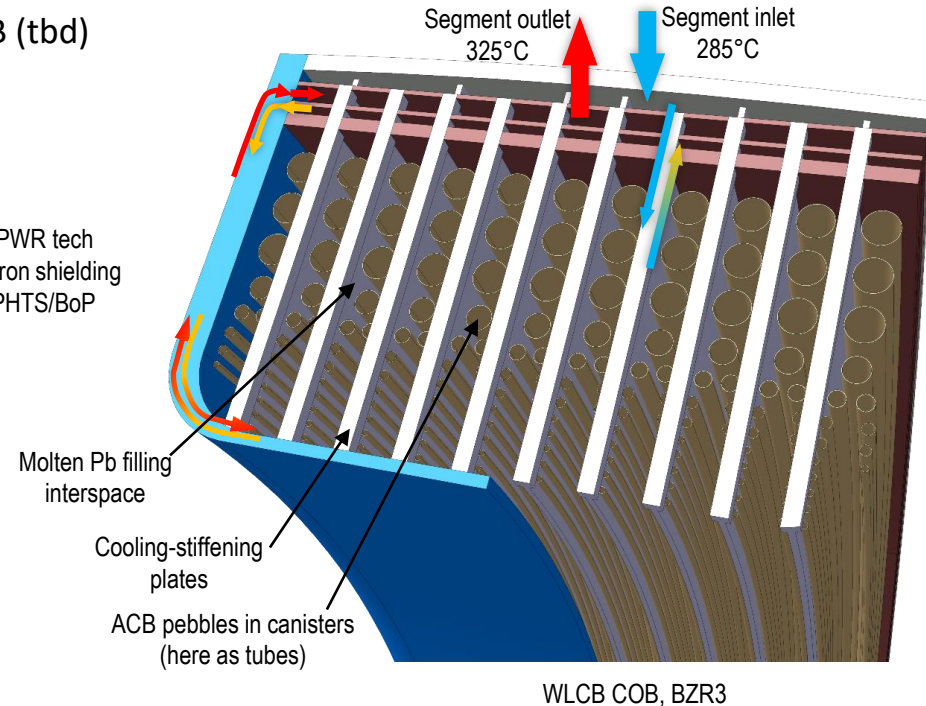
Trade-off between HCPB and WCLL:

- Mitigate n-shielding issues, n-mult. tech. and high costs of HCPB
- Mitigate T-permeation issues and tech. risks on PbLi TER
- Reduce dependency on anti-permeation barriers in BB (tbd)



Initial conceptual Idea

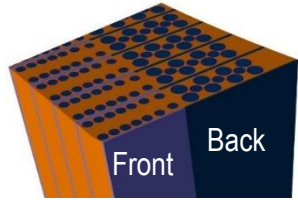
- PWR cooling (285-325°C @155bar)
- BZR: BZ and FW in series (as in WCLL-db)
- Purge gas: He + %H₂/H₂O @2bar (tbd)
- Radial cooling plates to withstand in-box LOCA
- ACB pebble beds for T-breeding in canisters (tbd)
- Molten Pb (n-multip) filling interspaces of BZR





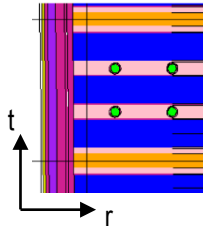
5. WLCB: Neutronics campaign

- ACB in poloidal configuration

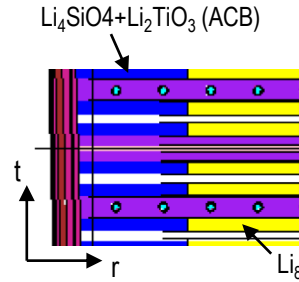


v7. CP: 50-25-25%
(E97-H2O-Pb)
Front & Back zones
TBR= 1.098

- ACB in radial configuration

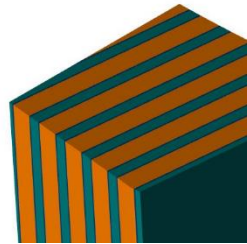
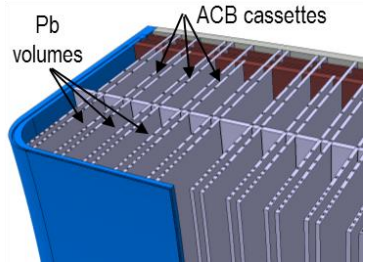


Poloidal cooling tubes
ACB, Li-6 90%, PF=64%
TBR=1.11

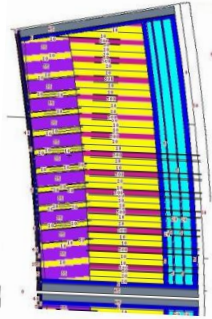
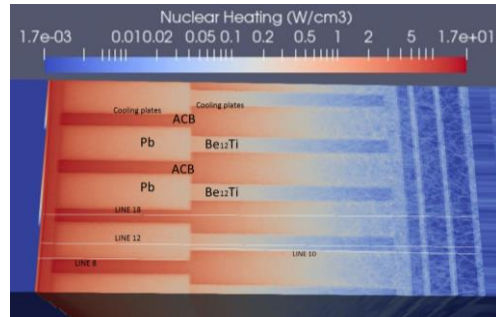
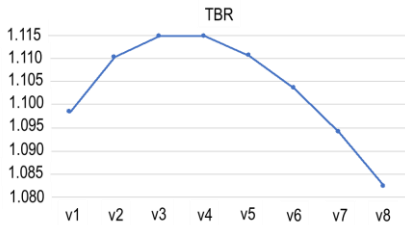


Poloidal cooling tubes
ACB + LOP, Li-6 90%, PF=64%
TBR=1.14
(D₂O coolant: TBR=1.17)

- ACB and NMM in cassettes



Rad-pol ACB cassettes
CP: 50-25-25% (H2O-st-Pb)
ACB, Li-6 90%, PF=64%
TBR=1.115

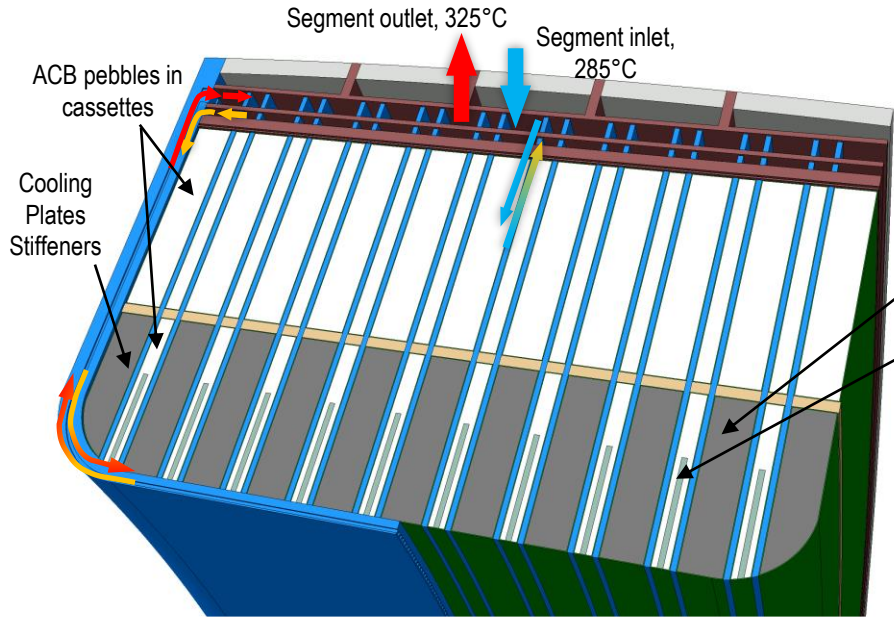


Front & Back zones
CP: 50-25-25% (E97-H2O-Pb)
ACB, Li-6 90%, PF=64%
TBR=1.137
ACB+LOP, Li-6 90%, PF=64%
TBR≈1.168

- Conclusions

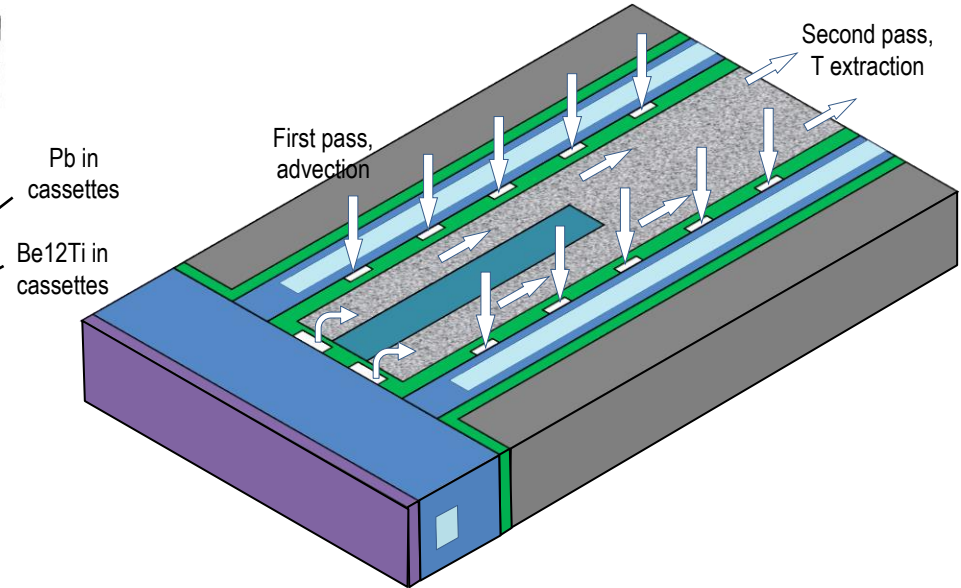
- Neutron economy in radial configuration better, more flexible
- Pb not efficient after 200mm: studies filling it with ACB or multiplier/reflector
- Addition of high Li density Li8PbO6 ceramics (LOP) in cold (back) BZ region can be key to add margins and/or reduce Li-6 % enrichment

5. WLCB: Matured conceptual design



WLCB COB, BZR3

- First pass: high speed purge gas through fins
- Second pass: purge gas low speed sweeping ACB pebble bed



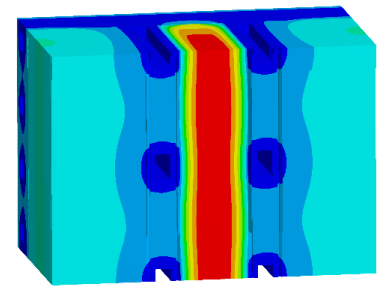
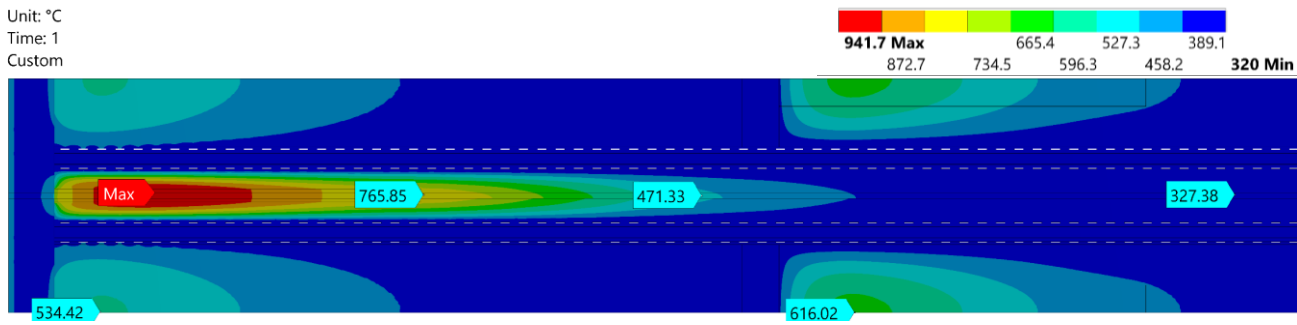
• Characteristics:

- All functional material enclosed in cassettes (no segment pipe for Pb),
- Finned contact with purge gas flow through interspace: same idea as WCLL-db to mitigate T permeation issue & leak detection method
- Finned contact may faster pressure relief after in-box LOCA, maybe lower design pressure of the segment
- R&D need: thermal management of BZ through finned contact needs to be qualified by testing

5. WLCB: Thermo-mechanics und Thermo-hydraulics

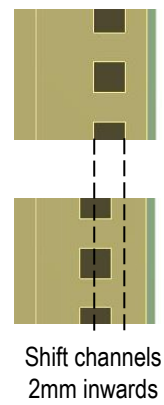
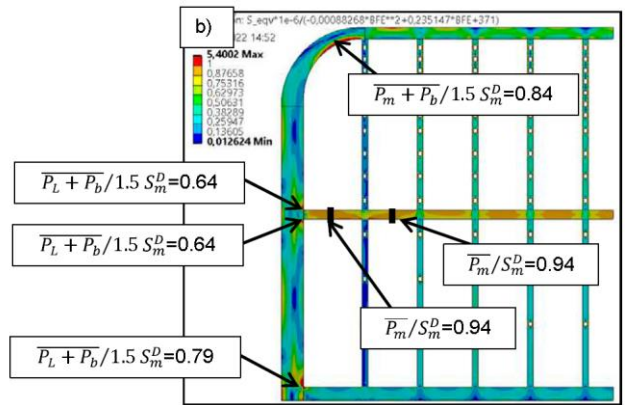
Simplified TH on matured WLCB design (cassettes)

Unit: °C
Time: 1
Custom

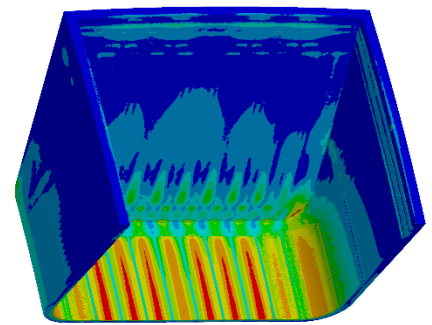
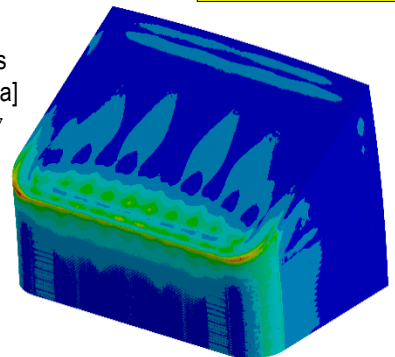
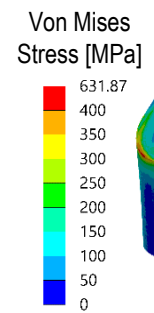


TM analyses on first WLCB design (tubes)

- Dimensioning the BZ key structures (CPs, toroidal stiffener) and upper cap region (NO and OP)



S. Giambrone, PS3-31



5. WLCB: Tritium transport analyses



Simplified T-transport analyses

Permeation rates to water circuits (g/d)	WCLL reference (PRF = 1)	WCLL-DB						WLCB tubes	WLCB cassettes	
		Stagnant purge gas	Flowing purge gas (\dot{m}_{HCPB})					Flowing purge gas (\dot{m}_{HCPB})	Flowing purge gas (\dot{m}_{HCPB}) 2 pass flow (gaps + CB)	
		495 °C	495 °C	330 °C	495 °C	330 °C				
		Perfect contact fins	Perfect contact fins	Perfect contact fins	No contact fins	No contact fins	No contact fins, 10x \dot{m}	Direct contact p.g. from TER	No contact fins p.g. from TER	No contact fins Pure p.g. supply
Water tubes	44.18	27.668	7.071	2.194	8.2E-02	2.5E-02	2.4E-03			
Feeding manifolds	--	4.878	9.0E-01	2.7E-01	9.3E-03	2.8E-03	2.7E-04			
First wall	2.38	4.330	5.2E-01	1.4E-01	5.3E-01	1.3E-02	1.3E-02			
Back wall	--	5.8E-01	2.2E-02	1.1E-02	3.5E-03	1E-02	1E-02			
Total	46.56	37.458	8.515	2.614	6.3E-01	1.7E-01	1.5E-01	7.35E-01	5.40E-01	tbd

Conclusions:

- WLCB permeation similar to HCPB, $\approx 1/100$ reference WCLL => less dependency on high performant (PRF) coatings

E. Carella, P3C3

- T concentration in purge gas at the inlet of WLCB is key to further improve T permeation figure

3. WLCB: Reliability/FMEA analysis



■ Summary: Scenarios with highest yearly failure rate (FR)

- Most scenarios show low yearly FR $<10^{-2}$, but some cases (table below) requires attention
- For first time, a design keeps yearly FR $< 10^{-1}$ for all failure modes => potential to meet availability targets

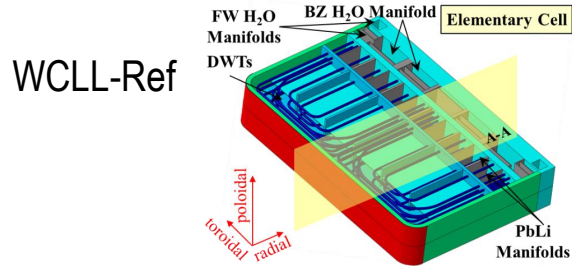
Failure and element	WCLL-Ref			WCLL-DB			WLCB			Consequence
	FR min [1/y]	FR max [1/y]	Multip.	FR min [1/y]	FR max [1/y]	Multip.	FR min [1/y]	FR max [1/y]	Multip.	
Leak/rupture F/T pipe PbLi	4.91E-02	4.09E-01	416	6.56E-02	5.47E-01	416	N/A			In-VV leak
Leak/rupture of poloidal welds between LiPb-BP and FW	2.57E-01	3.69E+00	72576	N/A			N/A			In-box LOCA
Leak of the LiPb-BP double welds	2.10E+00	2.33E+00	354816	N/A			N/A			In-box LOCA
Leak/rupture of pol. and tor. welds in the LiPb outlet manif.	2.34E+00	3.36E+01	661248	3.03E-04	4.34E-03	64	N/A			Bypass PbLi
Leak/rupture of weld of water pipes with water feed in/out feeder manifold halves	sw	N/A		3.62E-01	5.19E+00	76544	N/A			In-tube LOCA
	dw	N/A		3.62E-02	5.19E-01					
Leak/rupture weld of purge gas pipes with purge gas in/out feeder manifold halves	sw	N/A		3.62E-01	5.19E+00	76544	N/A			In-tube leak PbLi
	dw	N/A		3.62E-02	5.19E-01					
Leak/rupture of purge gas feeder manifold	N/A			2.78E-01	7.41E+00	11008	N/A			In-tube leak PbLi
Leak/rupture of purge gas chamber in FW	N/A			1.10E+00	2.94E+01	10000	N/A			In-tube leak PbLi
Leak/rupture of purge gas poloidal tubes	ST	N/A		3.62E+00	9.66E+01	38272	N/A			In-tube leak PbLi
	DWT	N/A		2.11E-03	3.62E-02					
Leak/rupture weld connection of the manif to the manif. from next breeder zone region	sw	N/A		2.72E-02	3.25E-01	4800	1.97E-02	2.82E-01	4160	In-tube/box LOCA
	dw	N/A		2.72E-03	3.25E-02		1.97E-03	2.82E-02		
Loss of structural integrity of the purge gas chamber	N/A			N/A			4.38E-01	4.38E-01	400	In-VV leak p.g.
Leak/rupture of structural weld of water manif. to the water CPs	N/A			N/A			5.75E-02	8.24E-01	12160	In-box LOCA



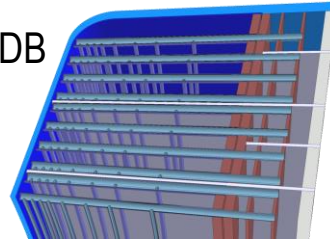
1. Reference BB concepts & motivation to explore variants
2. Exploring a WCLL variant: the WCLL „double bundle“
3. Exploring a WCLL-HCPB hybrid variant: WLCB
4. **Summary and Outlook**



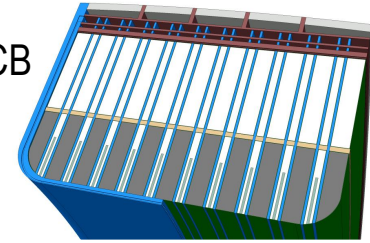
4. Summary and Outlook



WCLL-DB



WLCB



■ Summary

- WCLL-DB
 - First set of NK, TH/TM, T-transport and MHD studies prove potential, improving figures of WCLL
 - Reliability significantly improves when DWT (double welds) are introduced
- WLCB
 - Decision for cassette configuration: better NK, feasible TH/TM, T-permeation lower than WCLL-db
 - Worst yearly failure frequency for all critical failure modes $\sim 10^{-1}$

■ Outlook:

- WCLL-DB to be matured introducing DWT;
- Maturation of WLCB with cassettes
- Introduction of the WCLL-DB and WLCB in the WPBB baseline
- Optioneering among WCLL-ref, WCLL-DB and WLCB will follow for future reference selection





Backup slides

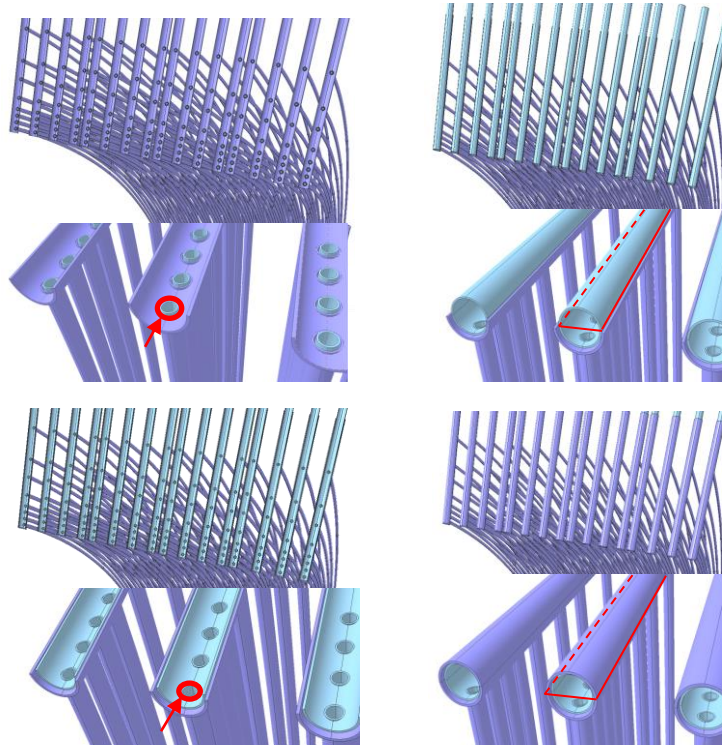
3. Exploring variants: WCLL “double bundle”



■ Summary 2021: Manufacturing and Assembly

Why is this important so early?

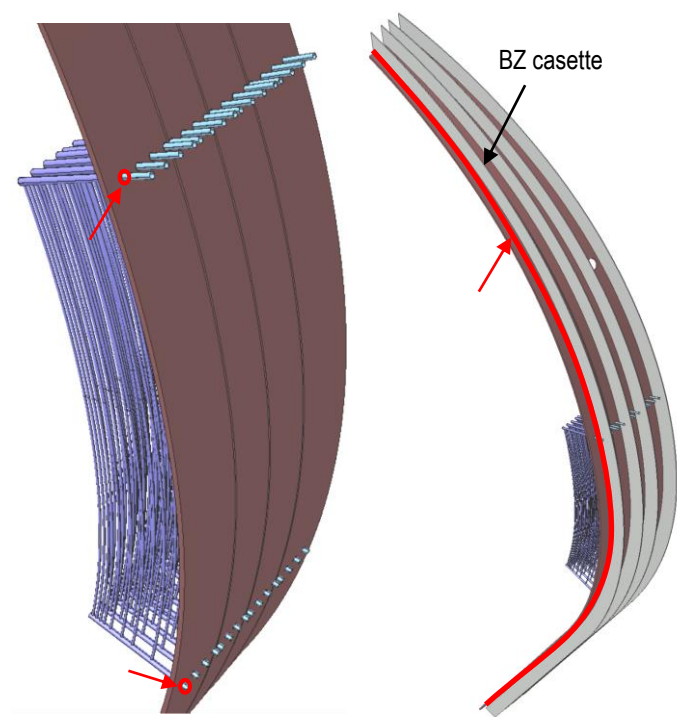
- Design for manufacturability
- Understanding architecture and estimation of number of welds is essential for RAMI analyses



1. Manufacturing of double bundle tubes (planar curvature)

2. TIG/laser weld of double bundle tubes feeders halves

3. TIG/laser weld of feeders halves



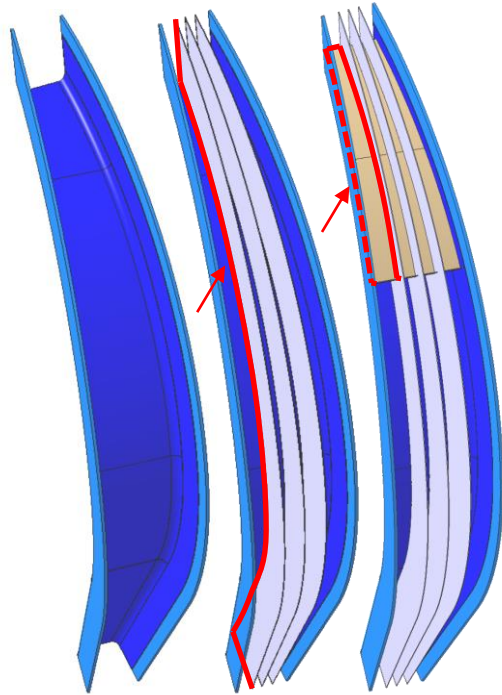
4. Orbital TIG weld of feeders to 1st BB manifold backplate to produce BZ cassettes

5. TIG weld of manifold stiffeners

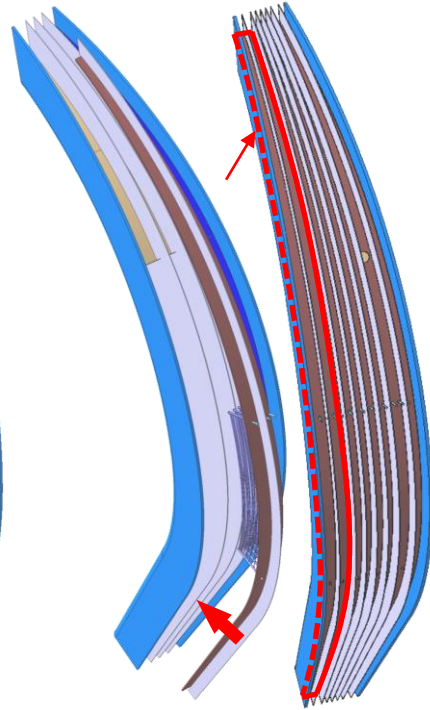
3. Exploring variants: WCLL “double bundle”



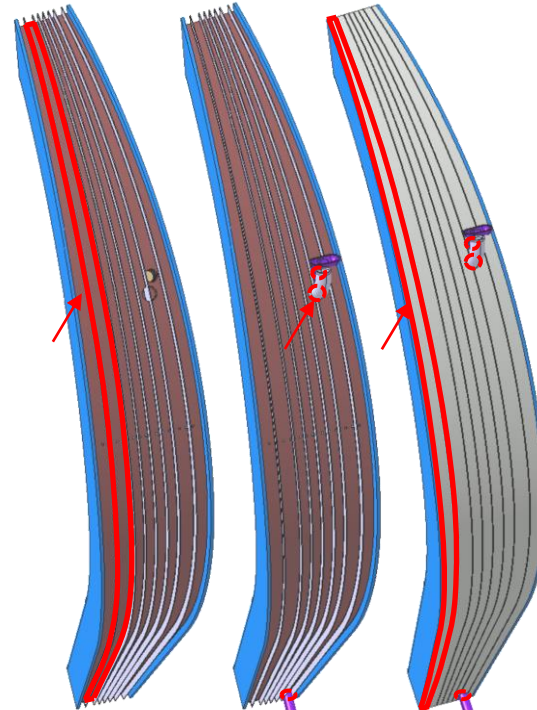
■ Summary 2021: Manufacturing and Assembly



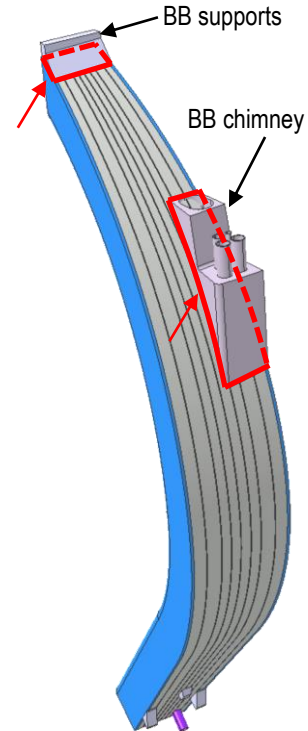
8. FW and caps production and assembly, with gas gap chamber and TIG/laser weld of stiffening plates (shown as continuous plates, but continuity not necessarily needed)



9. Insertion of BZ cassettes and TIG/laser weld to FW+caps assembly



10. TIG/laser weld of back manifolds, inlet and outlet segment pipes

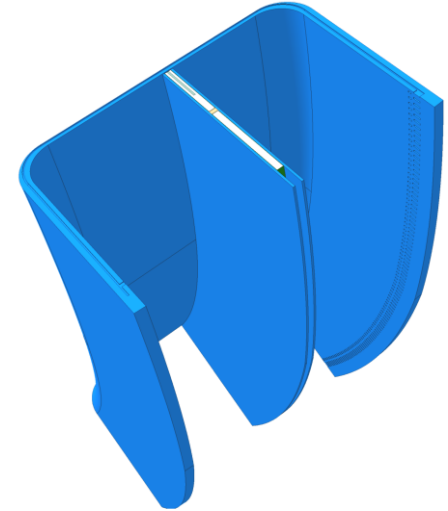
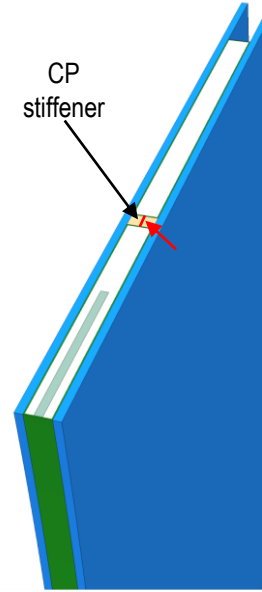
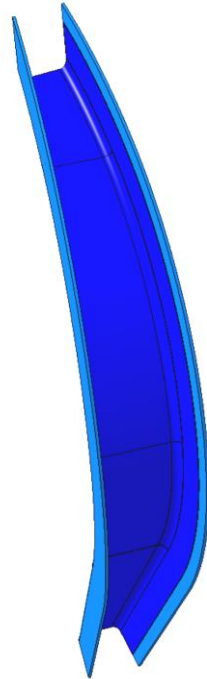
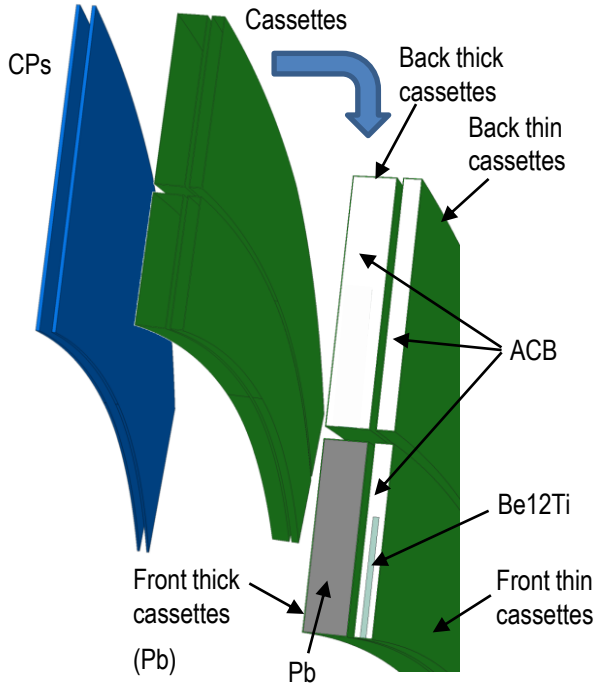


11. Chimney and segment supporting structures assembly TIG weld

WLCB: Manufacturing considerations



■ Manufacturing and assembly sequence



1. Production of BZ elements:

- Production of CPs and cassettes
- Production of ACB and NMM
- Cassettes filling

2. Production of FW in 5 parts and TIG/laser weld of caps (not shown)

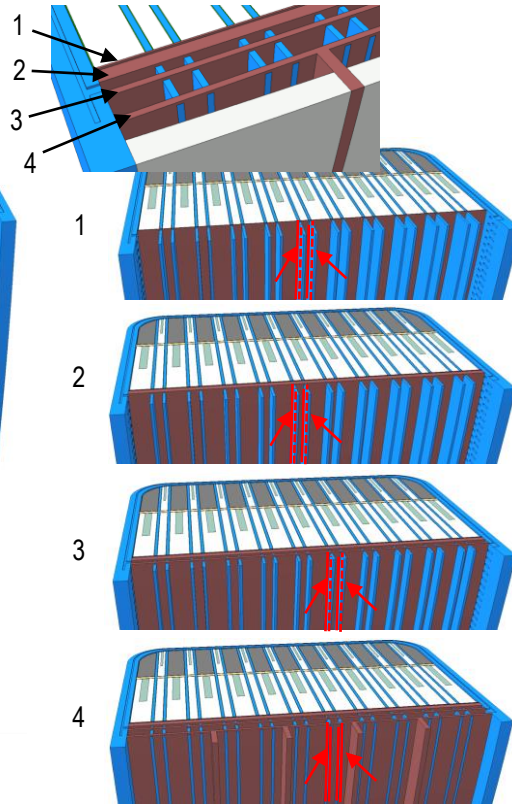
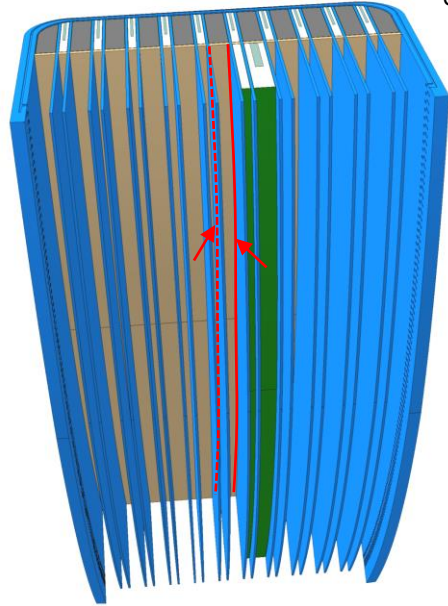
- ### 3. EB weld of CP thin stiffener of adjacent CP, insertion of thin front and back cassettes

- ### 4. Weld of CP with thin cassettes to FW

WLCB: Manufacturing considerations

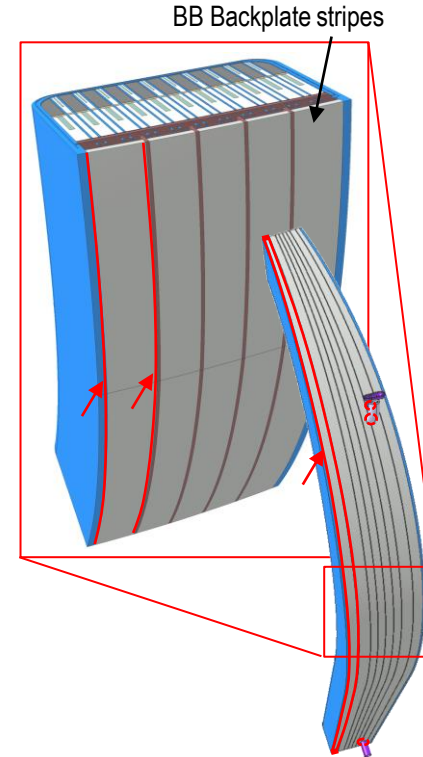


■ Manufacturing and assembly sequence

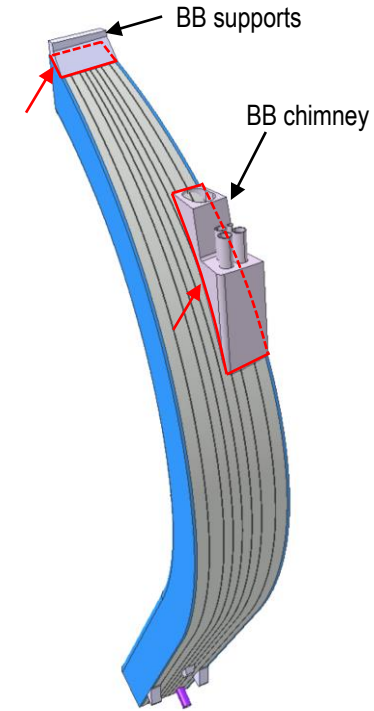


3. Insertion of front thick cassettes, weld of thick stiffener stripes and insertion of back thick cassettes

4. Insert and weld manifold plates. Only welds in 4 act against in-box LOCA.



5. TIG/laser weld of BB backplates to manifold stiffeners, FW & caps. Welds act against in-VV LOCA



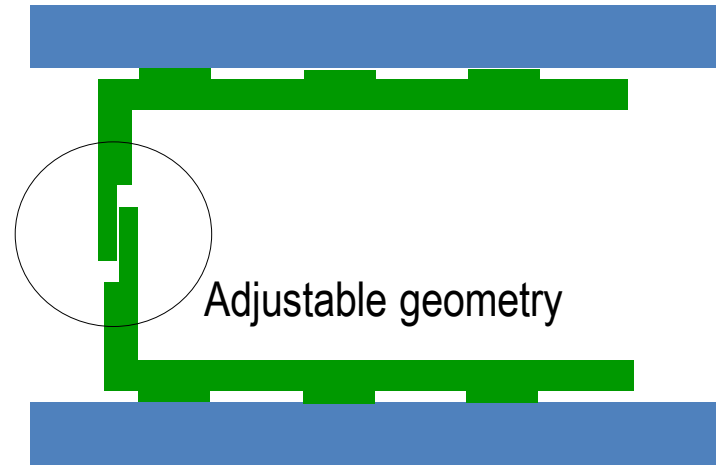
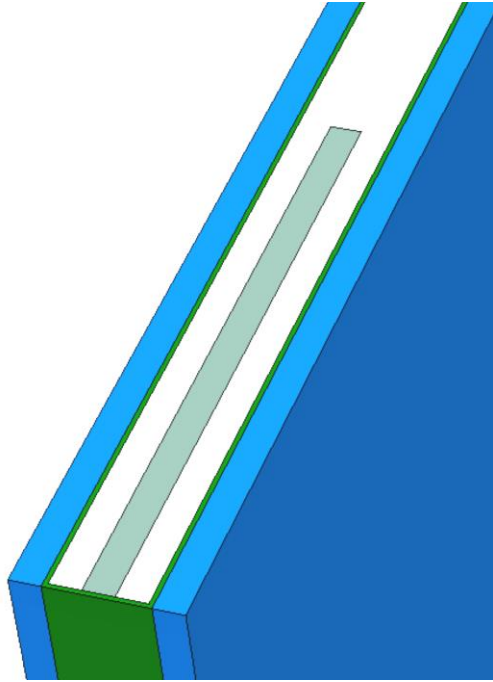
6. TIG weld of chimney (welds act against in-VV LOCA) & BB supports to BB backplate.

WLCB: Manufacturing considerations



- A note on the manufacturing of the cassette

The cassette could implement a feature to allow to adjust its toroidal thickness to the gap between cooling plates



WLCB: Relevant infos for reliability analyses



- Preliminary design specifications (v1.1, Pb also in cassettes, **preferred option**)

Component	COBS	LOBS/ROBS	LIBS/RIBS	Reactor total
FW	1	1	1	$(1 \times 3 + 1 \times 2) \times 16 = 80$
FW channels	$15000 / 24 = 625$	$15000 / 24 = 625$	$15000 / 24 = 625$	$625 \times 5 \times 16 = 50000$
CP channels ^a	$20 \times 5 \times 8 = 800$	$20 \times 5 \times 8 = 800$	$14 \times 5 \times 8 = 560$	$(800 + 2 \times 800 + 2 \times 560) \times 16 = 56300$
Welds against p-g leak in Pb ^b	$(11 + 33) \times 5 = 220$	$(11 + 33) \times 5 = 220$	$(8 + 24) \times 5 = 160$	$(220 + 2 \times 220 + 2 \times 160) \times 16 = 15680$
Welds against in-box LOCA ^c	$(20+5) \times 5 = 125$	$(20+5) \times 5 = 125$	$(14+5) \times 5 = 95$	$(125 + 2 \times 125 + 2 \times 95) \times 16 = 9040$
Welds against in-VV LOCA ^d	$5 + 2 + 5 = 12$	$5 + 2 + 5 = 12$	$3 + 2 + 5 = 10$	$(12 + 2 \times 12 + 2 \times 10) \times 16 = 896$

^aCP channels $20 \times 5 \times 8 = 800$

20 = number of CPs in a BZR

5 = number of BZR

8 = number of cooling channels per CP (assuming a reasonable water speed of <4m/s, channels could be halved to get a max. water speed of ≈7m/s)

^bWelds against p-g leak in Pb $(11+11+20) \times 5 = 240$

11 = welds of BZ stiffeners to CPs at Pb region

33 = welds of adjacent CP collectors & distributors & CP poloidally at FW front side

5 = number of BZR

In this version there are no such welds because Pb is confined in cassettes

^cWelds against in-box LOCA $(20+5) \times 5 = 125$

20 = countour welds of CPs

5 = 1 external contour of backplate + 4 separation between BZR

20 = welds of adjacent CP collectors & distributors

5 = number of BZR

^dWelds against in-VV LOCA $5 + 2 + 5 = 12$

5 = poloidal closure stripes forming the back plate

2 = caps

5 = FW BZR

WCLL-db: Summary



Basic preliminary design specifications

	WCLL-db
Coolant operating temperature [°C]	285 – 325
Coolant operating pressure [MPa]	15.5
Plant coolant mass flow [kg/s]	7450
Plant purge gas mass flow [kg/s]	0.5
Coolant feeding pipes, OB	Inlet: 1x DN200
	Outlet: 1x DN200
Coolant feeding pipes, IB	Inlet: 1x DN200
	Outlet: 1x DN200
Purge gas feeding pipes	Inlet: 1x DN80
	Outlet: 1x DN80
Pb feeding pipe	TBD
Number of BZR per segment	(Tentative 1x-DN80) 5
Number CPs	≈ 7040
Number coolant feeding pipes	≈ 160
Number purge gas feeding pipes	≈ 160
Reactor inventory CB (ACB / mix ACB & Li8PbO8)	539.2 / 1772.8 ton
Reactor inventory Be12Ti	53.2 ton
Reactor inventory Pb	2470 ton
Reactor inventory Steel	2978 ton

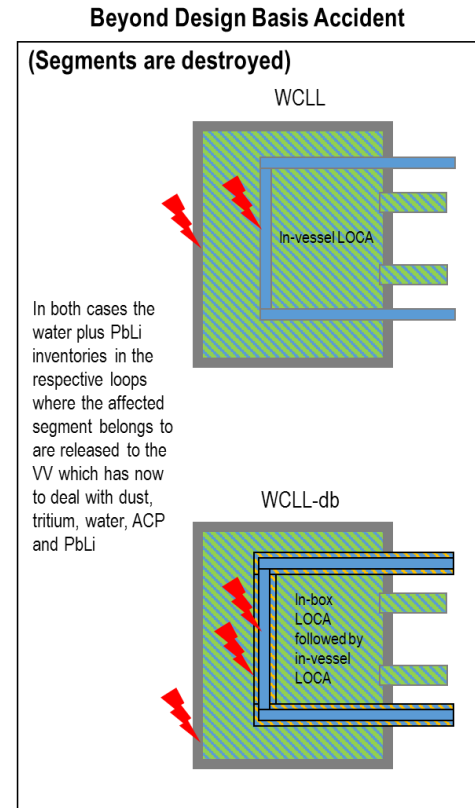
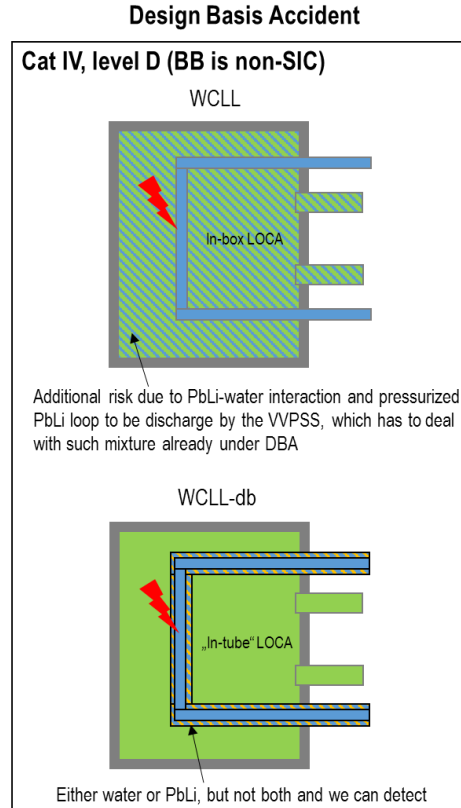
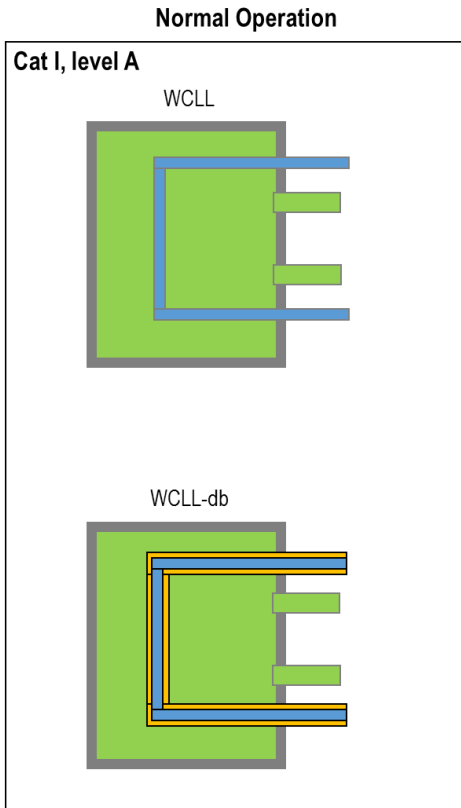
Inventory does not take into account cut outs in segments. These can be significant specially at the OB (average whole BB -10%)

Material	Segments inventory [ton]		
	COB	ROB/LOB	RIB/LIB
CB (KALOS / mix ACB & Li8PbO8)	8.5 / 28	7.4 / 24.4	5.2 / 17
Be12Ti	0.85	0.74	0.52
Pb	39	34	23.7
Steel	47	41	28.6
Total (per segment)	95.4 / 114.9	83.1 / 100.1	58.0 / 69.8

3. Exploring variants: WCLL “double bundle”



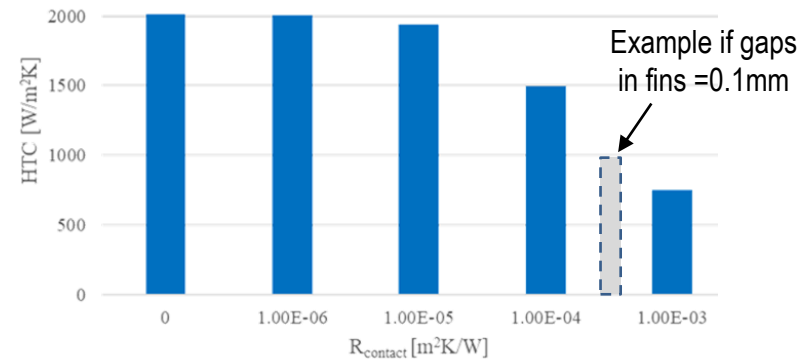
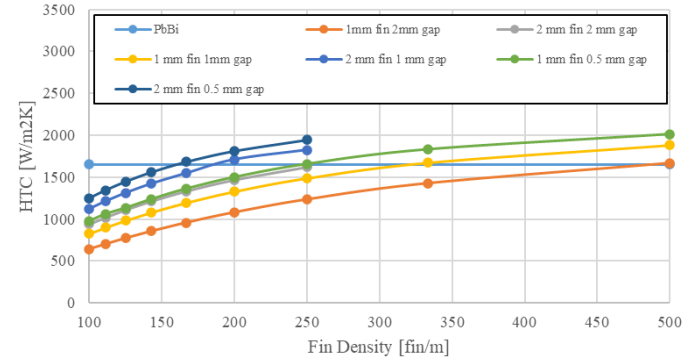
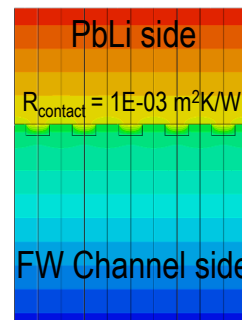
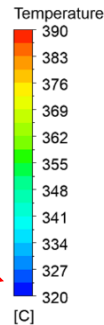
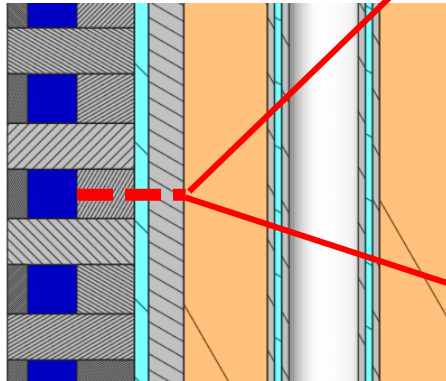
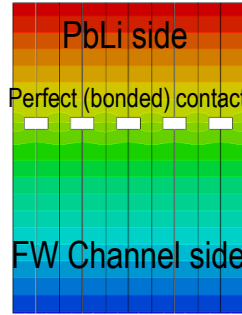
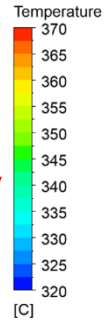
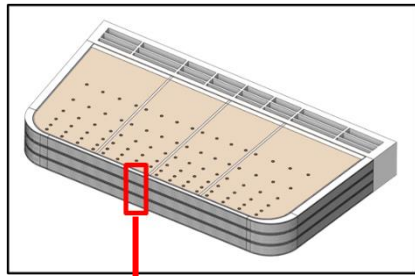
- Initial work 2022: Workshop with Safety Office on WCLL-DB vs. WCLL Safety and Licensing Case



TH of WCLL-db with gas gap



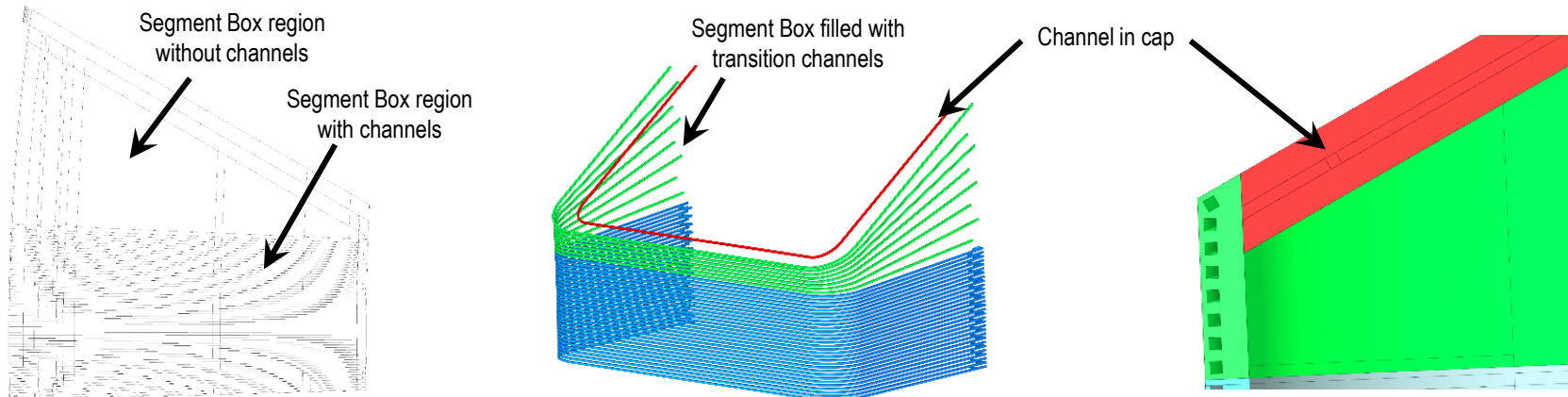
- Parametric study on HTC_{global} for the FW interlayer changing fin density and density and gap thickness
- Each HTC_{global} compared to the case with PbBi, He interlayer assumed adiabatic
- Sensitivity analysis also to evaluate the thermal contact resistance, considering the presence of He mitigating the HTC reduction.



5. WLCB: Caps Thermo-mechanics



- Design of a feasible WLCB cap
 - Caps: segment endings close to ports
 - Historically problematic due to non-smooth transition of poloidal to radial plates with different thickness requirements and difficulty to place stiffeners => large bending stresses and stress concentration
 - 3D FEM set-up and tested, temperature dependent material properties
 - Coolant flow (CPs and Seg. Box in series) modelled with ANSYS «thermal fluids» feature
 - automatically calculating the mixing temperatures in the CPs collectors and Seg. Box inlet manifold.
 - Iterative approach has allowed characterizing the fluid to obtain the Seg. Box outlet coolant average temperature in the whole BZR1 equal to ~ 325 °C, integrating analysis results with analytical estimations.



5. WLCB: Caps Thermo-mechanics



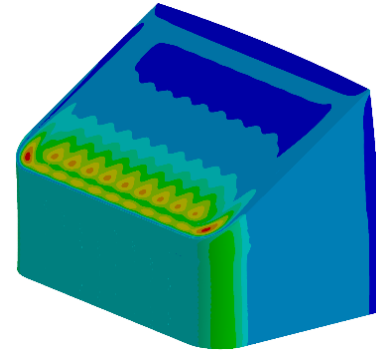
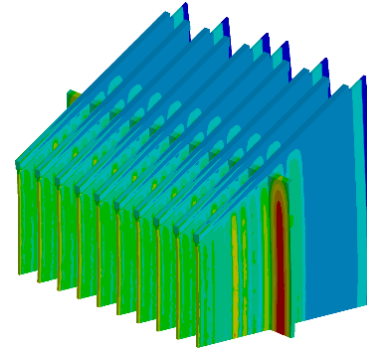
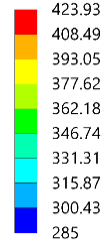
■ Thermal field

- Structural elements (CP and Seg. Box) below limits
- BZ: temperatures beyond 550°C within ACB tubes
- Revision of BZ needed after neutronics work finishes

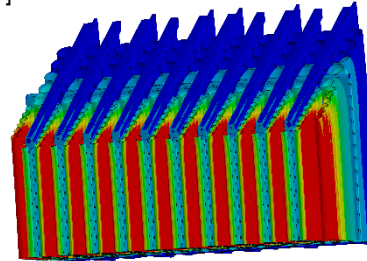
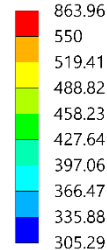
■ Stress field

- TM analyses in normal operation and over-pressurization (17.8 MPa)
- RCC-MRx criteria fulfilled with large margins in segment box and top cap, problematic regions limited to collectors and manifold

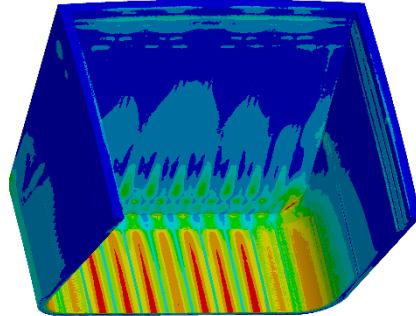
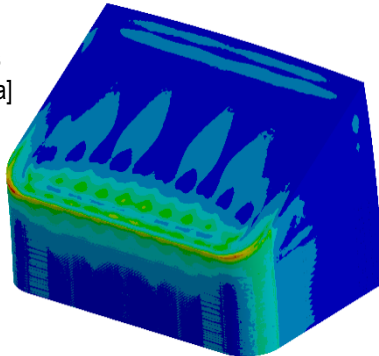
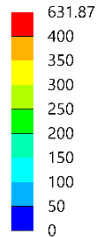
Temp. [°C]



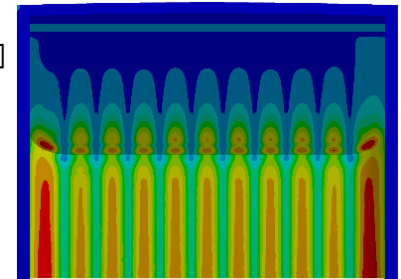
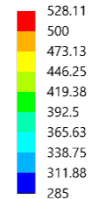
Temp. [°C]



Von Mises Stress [MPa]



Temp. [°C]



3. Exploring variants: WCLL “double bundle”



■ Initial work 2022: Safety and Licensing Case, Workshop with Safety Office

- Fundamental safety aspects of WCLL-DB vs. WCLL
 - DB interlayer can be used as monitoring point for leaks, early detection => safety characteristic to bring plant to safe state
 - LBB principle: Small LOCA leak detected before larger LOCA (this may help for event categorization, i.e. small vs. Large LOCA)
 - „In-tube“ LOCA avoids H₂O + PbLi and PbLi loop pressurization + release to VVPS regardless of the break size under DBA
 - DB: 2-layer redundant system (WCLL also a 2-layer redundant system, 2nd layer is the box itself) => rupture of the DB or box are extremely unlikely events => currently not postulated

■ 2021 and 2022: Workshops with WPBB and implementation of feedback

- Points regarding safety considerations
 - WCLL-DB based on assumption that „DB“ excludes LOCA, but not been demonstrated
 - „DB“ failure would be a common failure mode of a redundant system: if working with PbBi as interlayer, after in-tube LOCA water contacts PbBi and series of pressure pulses will occur => this triggers series of pressure waves in PbBi that may overcome water pressure => „DB“ failure => in-box LOCA in DBA possible => DB design assumption invalid, *but...*
 - If interlayer empty (i.e. gas) => interlayer pressure = coolant pressure, no pressure waves => possible solution
 - Inner tube breaks, banging/whipping against outer tube occurs => „DB“ failure
 - With gas gap, fins are necessary => whipping mitigated



journal

J. Am. Ceram. Soc., 73 (6) 1740-43 (1990)

Investigation of Lithium Diffusion in Octalithium Plumbate by Conductivity and NMR Measurements

Satoshi Konishi, Hideo Ohno, Takumi Hayashi,* and Kenji Okuno
Japan Atomic Energy Research Institute, Tokai, Ibaraki, 319-41 Japan

Toru Matsuo

Department of Physics, Faculty of Engineering, Toyo University, Kawagoe, Saitama-ken 350, Japan

Diffusion of lithium ion and tritium in octalithium plumbate (Li₈PbO₆) was studied. The electrical conductivity of the polycrystalline pellets measured by the two-terminal ac method in the temperature range of 300 to 973 K was one of the highest among oxide lithium ceramics. The temperature dependence of the conductivity is consistent with the nuclear magnetic resonance of lithium-7 powder samples, suggesting that the temperature dependence of the diffusion of lithium consists of three regions in this temperature range. Preliminary measurements of the diffusion coefficient of tritium in neutron-irradiated Li₈PbO₆ powder were also carried out. The results and the

II. Experimental Procedure

(1) Sample Preparation

Samples of Li₈PbO₆ were synthesized by the solid reaction between Li₂O and PbO₂. Powders of Li₂O and PbO₂ were mixed at a ratio of 4 to 1 and agglomerated in dry argon atmosphere in a glovebox. The appropriate powder was pressed into cylindrical pellets of 6-mm diameter with a hydrostatic press at a pressure of 1.2 ton/cm². These pellets were loaded into a platinum crucible and were heated at 873 K in a dry oxygen stream for several hours. Reaction and sintering took place at the same time, and slightly yellowish sintered pellets

Fusion Engineering and Design 87 (2012) 482–485

Contents lists available at SciVerse ScienceDirect

Fusion Engineering and Design

journal homepage: www.elsevier.com/locate/fusengdes

ELSEVIER



Octalithium plumbate as breeding blanket ceramic: Neutronic performances, synthesis and partial characterization

S. Colominas^{a,*}, I. Palermo^b, J. Abellà^a, J.M. Gómez-Ros^b, J. Sanz^c, L. Sedano^b^a Universitat Ramon Llull, ETS Institut Químic de Sarrià, Electrochemical Methods Laboratory – Analytical Chemistry Department, Via Augusta, 390, 08017 Barcelona, Spain^b CIEMAT, Av. Complutense 22, E-28040 Madrid, Spain^c INED, Department of Nuclear Energy, c/Juan del Rosal 12, E-28040 Madrid, Spain

ARTICLE INFO

Article history:
Available online 30 January 2012

Keywords:

Blanket
Octalithium plumbate
Synthesis
XRD
TBR
Neutronic response

ABSTRACT

A neutronic assessment of the performances of a helium-cooled Li₈PbO₆ breeding blanket (BB) for the conceptual design of a DEMO fusion reactor is given. Different BB configurations have been considered in order to minimize the amount of beryllium required for neutron multiplication, including the use of graphite as reflector material. The calculated neutronic responses: tritium breeding ratio (TBR), power deposition in TF coils and power amplification factor, indicate the feasibility of Li₈PbO₆ as breeding material. Furthermore, the synthesis and characterization of Li₈PbO₆ by X-ray phase analysis are also discussed.

© 2012 Elsevier B.V. All rights reserved.

TRITIUM RELEASE BEHAVIOR FROM NEUTRON-IRRADIATED Li₈PbO₆

T. HAYASHI, S. KONISHI and K. OKUNO

Japan Atomic Energy Research Institute Tokai, Ibaraki 319-11, Japan

Received 21 April 1989; accepted 1 August 1989

Chemical behavior of tritium produced in octa-lithium plumbate (Li₈PbO₆) crystals by the ⁶Li(n, α)T reaction has been investigated. Nearly 100% of the tritium in the crystals existed in the T⁺ state. When the neutron-irradiated crystals were heated up to 1073 K under vacuum, almost all the tritium released in the chemical form of tritiated water (HTO(g)). The HTO(g) release rate was controlled by diffusion of tritium (T) in the crystals and the diffusion coefficient (D) determined in the temperature range from 580 to 670 K was

$$D = 1.1 \times 10^{-4} \exp\{-75.5(\text{kJ mol}^{-1})/RT\} \text{ cm}^2 \text{ s}^{-1}.$$

The observed tritium diffusivity in Li₈PbO₆ was the largest of the lithium-based oxide ceramics previously reported. This coincides with the fact that the diffusivity of lithium ion in crystals was the largest of these ceramics.



Fusion Engineering and Design

Volume 137, December 2018, Pages 243–256



First principles review of options for tritium breeder and neutron multiplier materials for breeding blankets in fusion reactors

F.A. Hernández P. Pareslavtsev

Show more

Add to Mendeley Share Cite

<https://doi.org/10.1016/j.fusengdes.2018.09.014>[Get rights and content](#)

Abstract

The current breeding blankets proposed in the different conceptual fusion power plants are based mainly on the use of Li₄SiO₄ and/or Li₂TiO₃ as tritium breeder and Be/Pb₁₂Ti as neutron multiplier or an eutectic Li₇Pb₃ as a hybrid tritium and