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Low Flux Test Module

Medium Flux Test Modules

High Flux

Test Module

Lithium Target

Assembly

# BLUME: A <u>blanket functional materials module</u> for the Helium Cooled Pebble Bed breeding blanket in IFMIF-DONES

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- Why test Breeding Blanket mockup in IFMIF-DONES?
- Testing goals of BLUME
- Initial design of BLUME
- Summary and outlook

### Breeding Blanket: a key system in any D-T fusion electricity device

 ${}_{1}^{2}D + {}_{1}^{3}T \rightarrow {}_{2}^{4}He + n + 17.6 \text{ MeV}$ 

Tritium breeding blanket is a nuclear component within vacuum vessel that surrounds the plasma like a blanket.

Deuterium (D) has a large natural abundance in the oceans.

Tritium (T) decays and has no natural abundance. About 55 g T is lost per year for 1 kg T.

1 GW fusion (thermal) power device consumption: **56** kg *T* per full power year (fpy).

Need to produce *T*, in order to be economically viable!





- Main functions of the Breeding Blanket:
- > tritium breeding => tritium self-sufficiency
- heat removal => electricity production
- shielding => protect magnets





# Why test Breeding Blanket mockup in IFMIF-DONES?

- Despite the importance of blanket, maturity level of breeding blanket is still very low.
- Feasibility concerns and uncertainties exist in all explored breeding blanket concepts.
- Significant research and development are needed to address these issues.



In European DEMO program, Helium Cooled Pebble Bed (HCPB) breeding blanket is under development.

We propose a <u>blanket functional materials module</u> (BLUME) for HCPB blanket to be tested in IFMIF-DONES.

### Neutron flux spectra and volume for testing in IFMIF-DONES



## Preliminary neutronics analyses to check neutron flux relevancy



Blue: Advanced ceramic breeder Li-pebbles Green: Titanium beryllide Magenta: Eurofer97 steel







Neutron flux density of HCPB BB in EU-DEMO



Neutron footprint: 400 x 400 mm<sup>2</sup>

#### Neutron flux density of HCPB mockup in IFMIF-DONES



# Testing goals of BLUME (<u>Blanket functional materials module</u>)

#### Heat Transfer Experiments

| $\smile$   | Goals  | To address heat transfer in realistic fuel-breeder pin geometry  |
|--|--|--|
|  | Measurements   | Temperature (breeder, coolant, and purge), purge pressure drop, and post-test examination of the breeder structure and gap dimensions  |
| (2)  | Neutronics Prediction Validation (Tritium Generation, Nuclear Heating, Activation) |  |
| <b>·</b>   | Goals  | To verify neutronics predictions for tritium breeding, nuclear heating and activation  |
|  | Measurements   | Post-test examination for activation, tritium inventory, and neutron fluence   |
| 3 Breeder/Structure Thermo-Mechanical Interactions |  | no-Mechanical Interactions   |
|  | Goals  | To study thermomechanical interactive effects on component behavior  |
|  | Measurements   | Temperature, stress, and post-test examination for gap size, cracking, sintering, settling, swelling, and other changes<br>Stress and strain in the structure, cracking and redistribution in the breeder, and overall deformation and failure modes |
| 4  | Tritium Behavior in Thermal and Flow Transients                                    |  |
| 0  | Goals  | To investigate the tritium inventory and permeation behavior during thermal and flow transients  |
|  | Measurements   | Measurements include temperature (breeder and coolant), coolant and purge tritium activity, and post-test examination for tritium inventory, cracking, or other changes in the solid breeder   |
| (5)  | Blanket Response to Coolant Transients   |  |
| $\checkmark$                                       | Goals  | To assess the effect of loss of flow or loss of coolant conditions on the fuel-breeder pin   |
|  | Measurements   | Measurements include temperature, stress, coolant pressure, flow rate, and post-test examination for deformation and failure   |

# Testing boundary conditions

- Helium coolant at 3.5 bar pressure to avoid pressurization in Test Cell
- To ensure relevant temperature field of HCPB blanket
  - ✓ Coolant inlet temperature: 350 380 °C
  - ✓ Coolant outlet temperature: 500 520 °C
  - ✓ Eurofer temperature: 300 550 °C
  - ✓ Advanced ceramic breeder Li-pebble temperature: 400 920 °C
  - ✓ TiBe12 temperature: 300 1000 °C
  - $\checkmark$  Mass flow rate range at 3.5 bar: 1 g/s 10 g/s









9-degree inclination of deuteron beam causes asymmetry in nuclear heating Total power: 1044 W

Tritium generation rate: about 4e-06 mg/s, 1 day T-production: 0.34 mg



# Structural analysis: Von-Mises Stress field [MPa]

Coolant-wetted surfaces: 3.5 bar pressure Body: temperature field Fixed at rear surfaces of BLUME



121 90 78.8 67.5 56.3 45 33.8 22.5 11.3 0.00385

Stress with pressure load

Stress with pressure and temperature field

#### Structure is robust enough to withstand the pressure and thermal stresses.



### ➤ Summary:

- Testing goals of BLUME are presented
- Initial design of BLUME is proposed
- Neutronics, thermal and structural analyses confirm the DEMO-relevancy

### ≻Outlook:

- The surroundings of the central pin will be replaced with reflecting materials (TiBe12 or graphite blocks)
- Interface with Test Cell of IFMIF-DONES will be established
- Measurement requirement of different test goals will be defined
- Ancillary systems for BLUME will be defined







