



Current status of IFMIF DONES project

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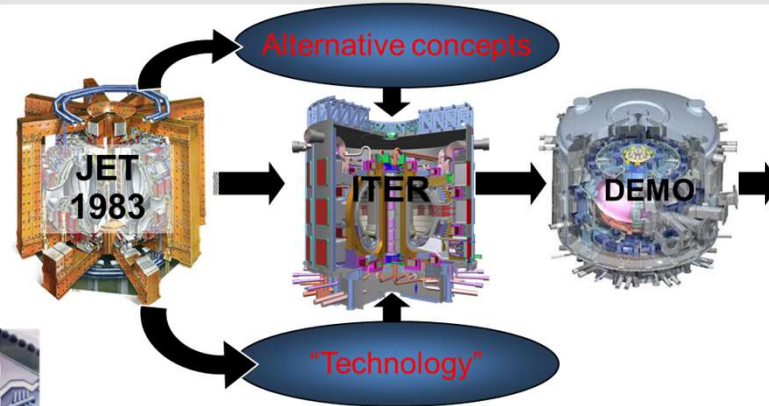
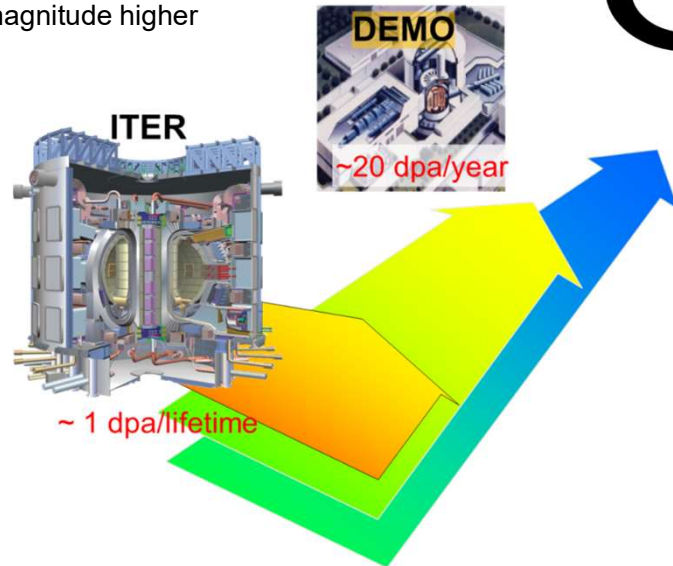
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- **Introduction**
- **IFMIF-DONES Facility Description**
- **IFMIF-DONES Experimental Capabilities**
- **IFMIF-DONES Status**
- **Summary**

Why DONES?



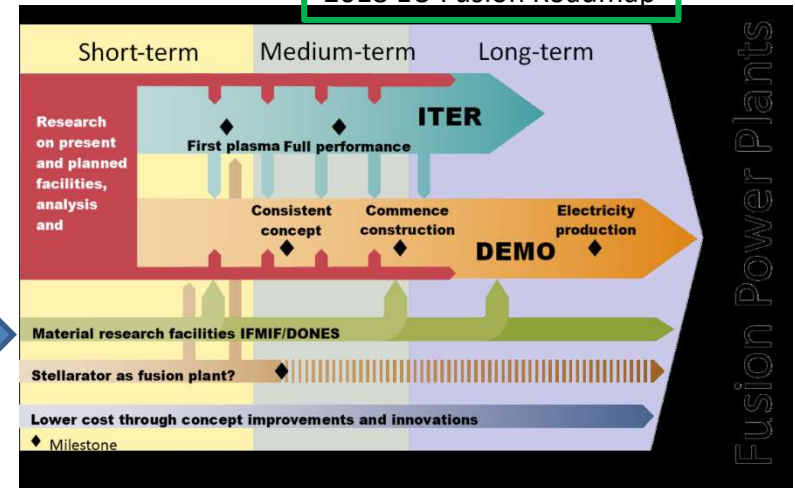
One of the main differences between ITER and DEMO is the radiation dose: at DEMO more that two orders of magnitude higher



The power plant

EU strategy towards fusion energy

2018 EU Fusion Roadmap

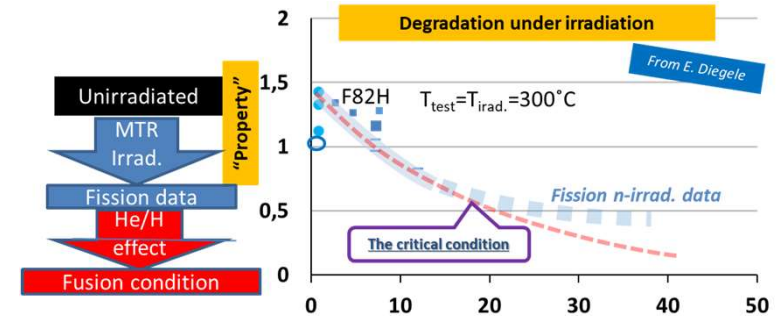


Fission irradiations

- Intensive and broad use of MTR (Material Test Reactors) fission irradiation: EU plans for 50M€ in the next decade
- Complementary irradiation modelling and verification (multi-ion beams)



To establish 1st step “best estimate” to perform engineering design

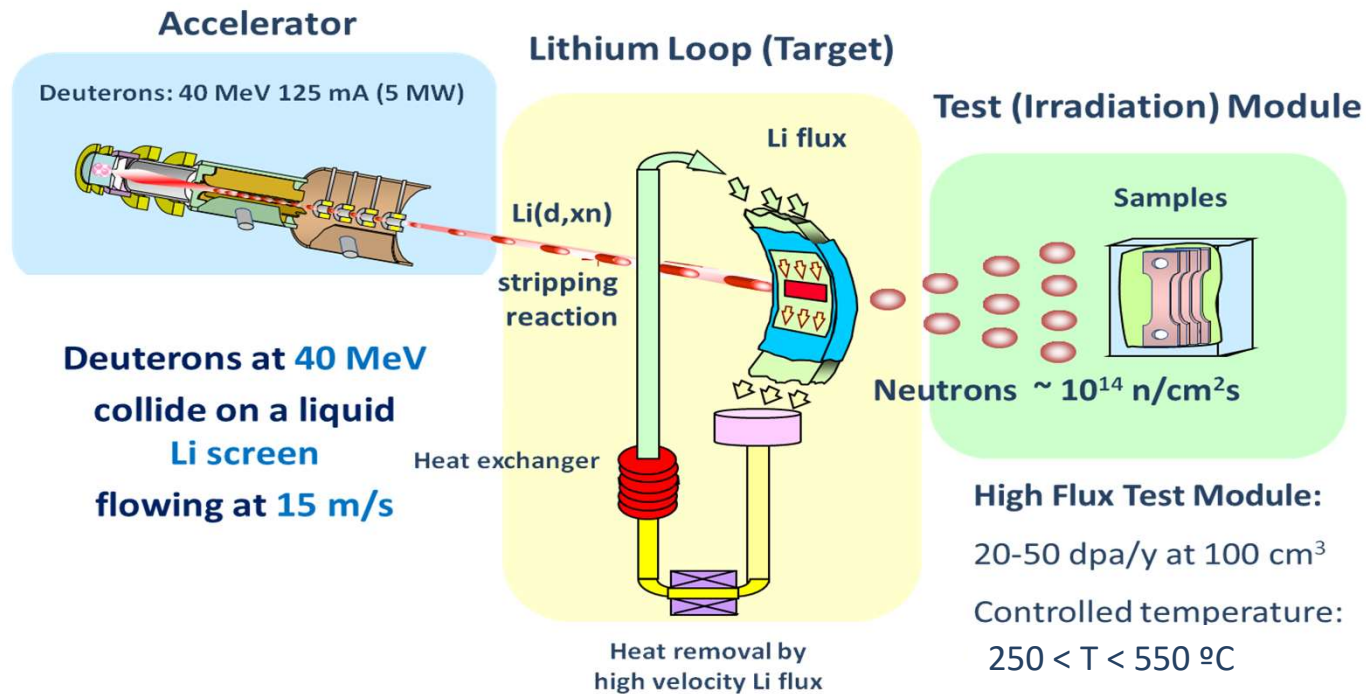


Fusion-like irradiations

- Mandatory: a dedicated facility for material qualification that best mimics 14Mev neutrons with reasonable irradiation volume, fluence, and optimized homogeneity in T with the objective to (finally) validate in-vessel materials

Based on the assumption that fusion-related effects will appear only at high dose (>10-20 dpa)

A fusion-like neutron source required for the qualification of the materials to be used in the EU DEMO

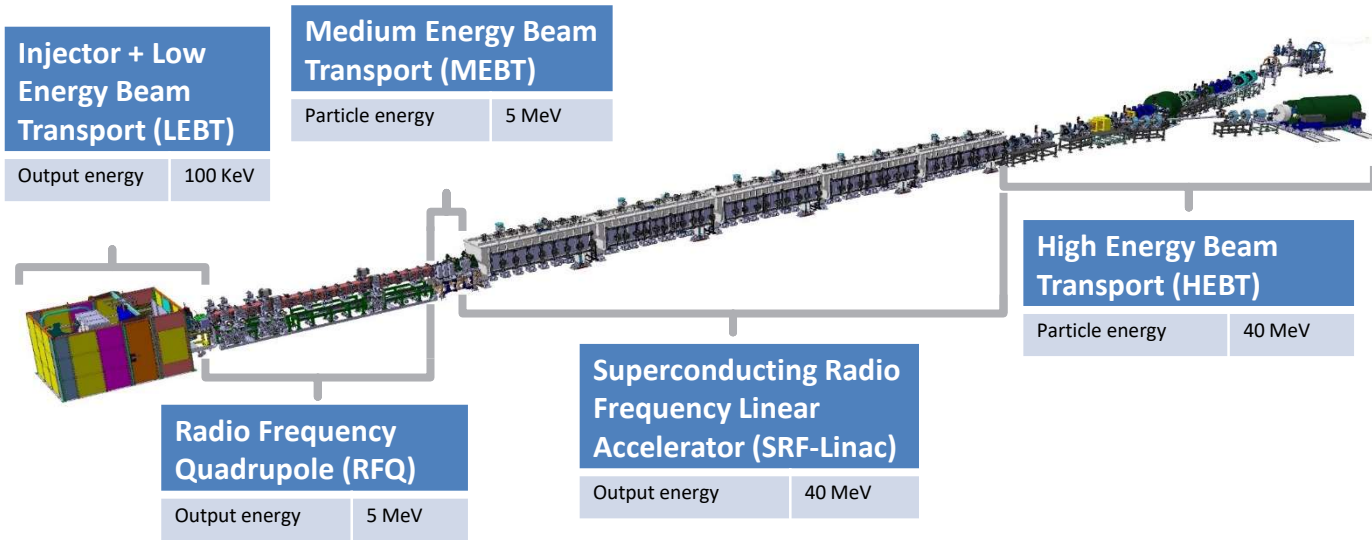
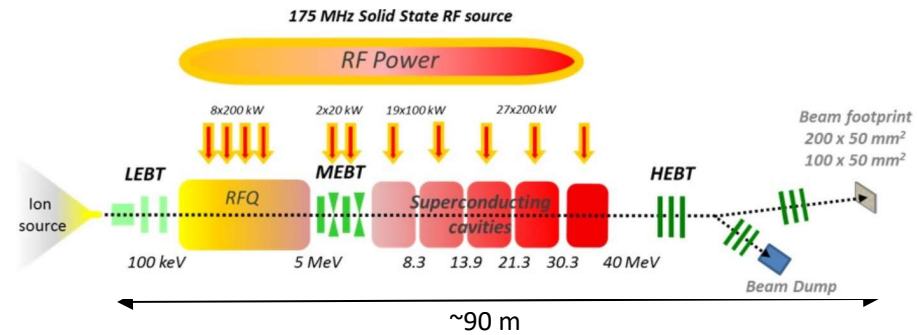


Identified as high priority in the EU Fusion Roadmap
Included in the ESFRI Roadmap as a EU strategic facility

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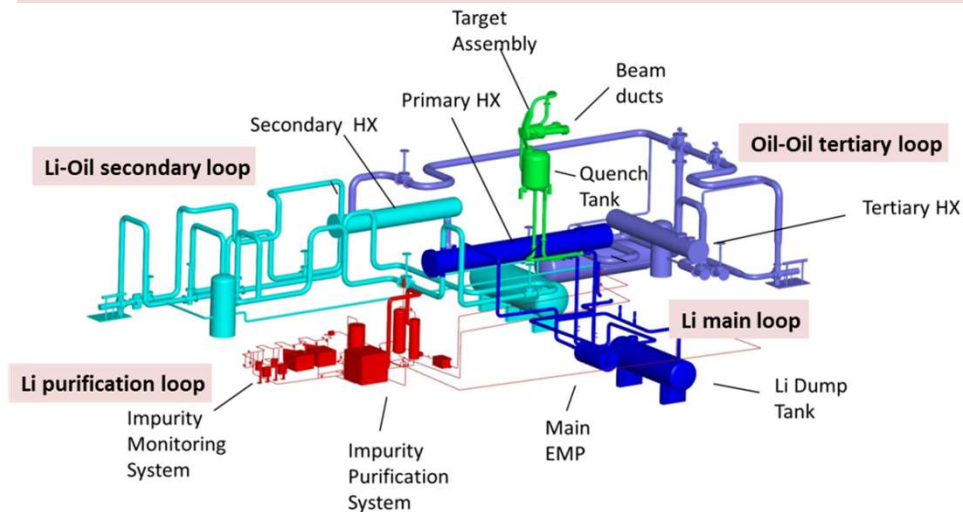
175 MHz, 5MW, 125 mA, CW, high availability: One of the more powerful accelerators in the world

Obtaining validation results from IFMIF-EVEDA: LIPAc Prototype (Rokkasho)



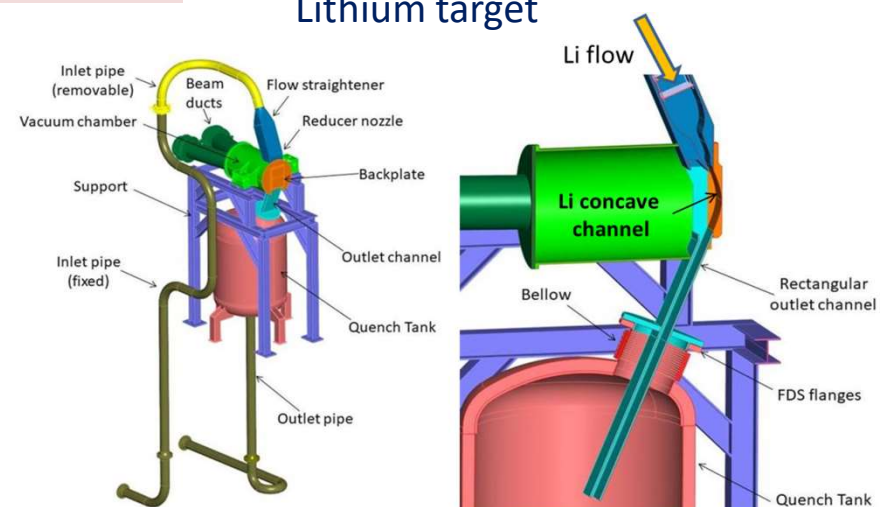
- Main involved technologies**
- RF
 - Cavities
 - Magnets
 - Mechatronics (Cu, Nb, Al,...)
 - Criogenics
 - Vacuum
 - Power supplies
 - Cooling technologies
 - Sensors and diagnostics
 - Control (hardware and software)

5 MW power handling, 15 m/s Li speed, remote handling maintenance
 Main requirements: Li flow stability and Li impurities control



Li volume $\sim 14 \text{ m}^3$ Li flow rate $\sim 100 \text{ l/s}$
 Temperature (inlet Target) $\sim 300 \text{ }^\circ\text{C}$

Lithium target

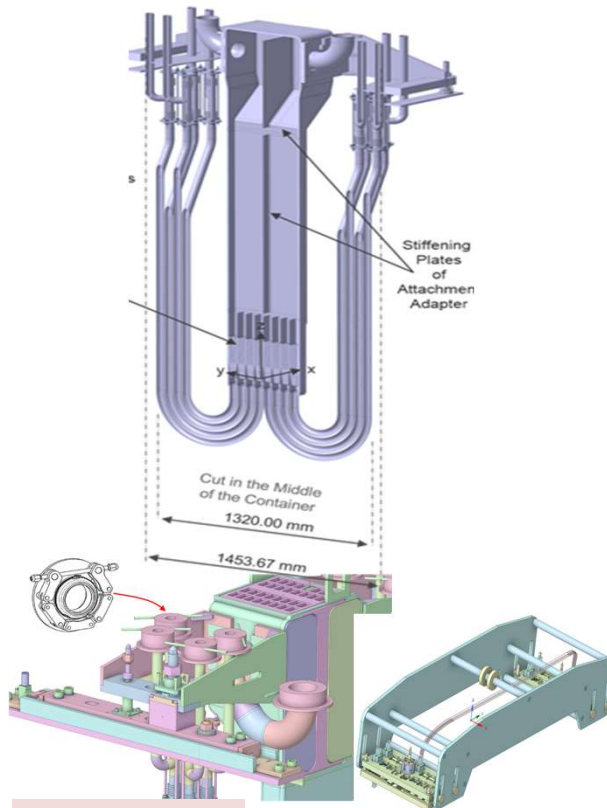


Jet thickness: $25 \pm 1 \text{ mm}$ Li flow speed: 15 m/s
 Chamber pressure: 10^{-3} Pa Heat flux: 500 MW/m^2

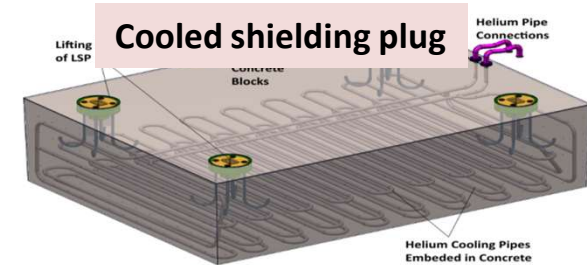
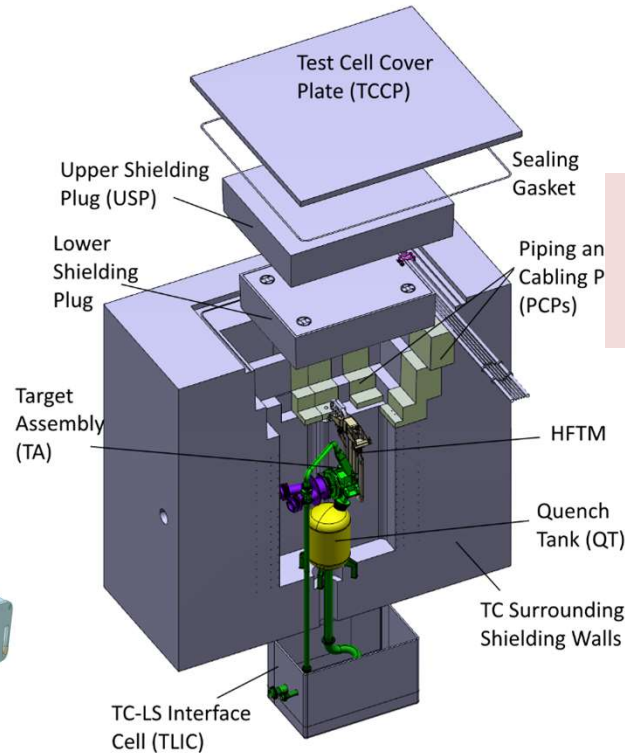
Main involved technologies

- Liquid metals (fluids, monitoring and purification)
- Complex cooling loops
- Diagnostics
- Remote handling maintenance
- Control (hardware and software)

Irradiation module



Connections



Main characteristics driven by the presence of neutrons and Li

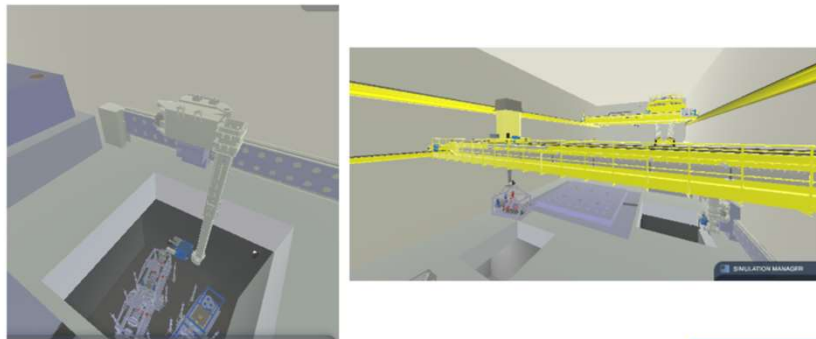
- Internal components cooling by He
- Remote Handling Maintenance required

Main involved technologies

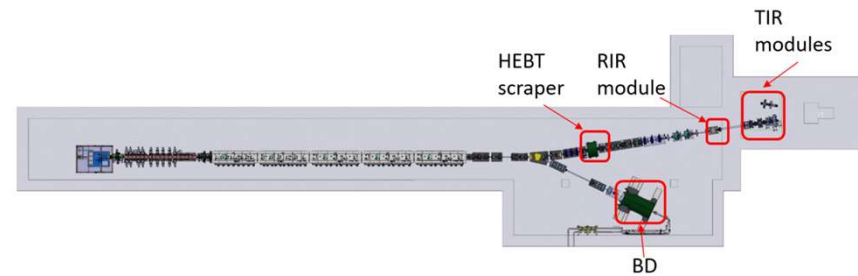
- Mechatronics
- He and water cooling
- He, Ar and water systems
- Shielding materials and technologies
- Remote maintenance
- Vacuum
- Diagnostics
- Control (hardware and software)

☐ Activated and contaminated equipment requires Remote Handling Maintenance

Access Cell: TIR & TC

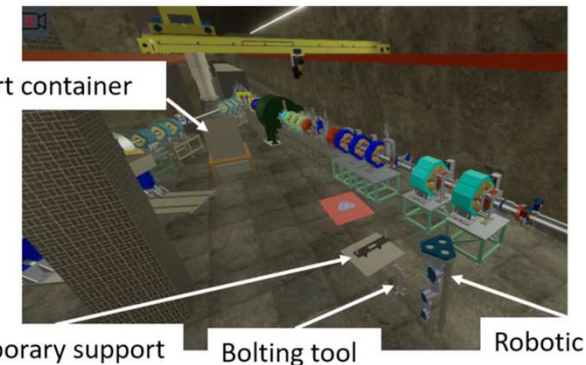
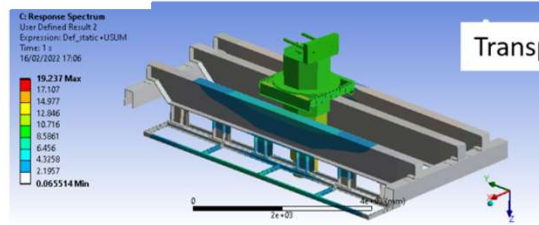


Accelerator



Main involved technologies

- Special cranes
- Telemanipulators
- RH tools
- Radiation monitoring



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• Materials qualification

Experiments to be developed in the irradiation area with the highest neutron flux are managed by specific irradiation modules that can be replaced (and modified) after each irradiation campaign

Present baseline design activities focuses on the High Flux Test Module (HFTM) for high-priority structural materials irradiation

Steel irradiation

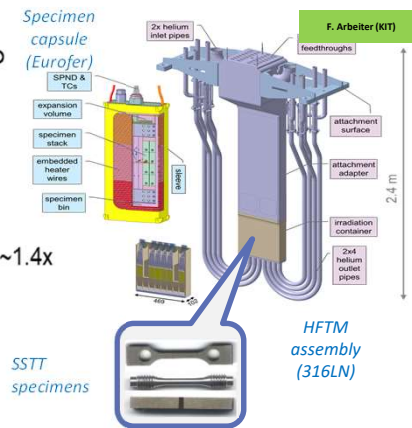
- 13-35 dpa/fpy up to 300 cm³ (22-50 dpa/fpy with two accelerators)
- 10-15 appmHe/dpa, 45-55 appmH/dpa.
- 250 – 550 °C, (~ 1000 specimens)

Copper irradiation (divertor heat sink)

- 5–30 dpa/fpy
- 6–8 appm He/dpa is (~DEMO), 48–50 appmH/dpa (~1.4x DEMO)
- >100°C, helium immersed specimens

Tungsten irradiation (armor)

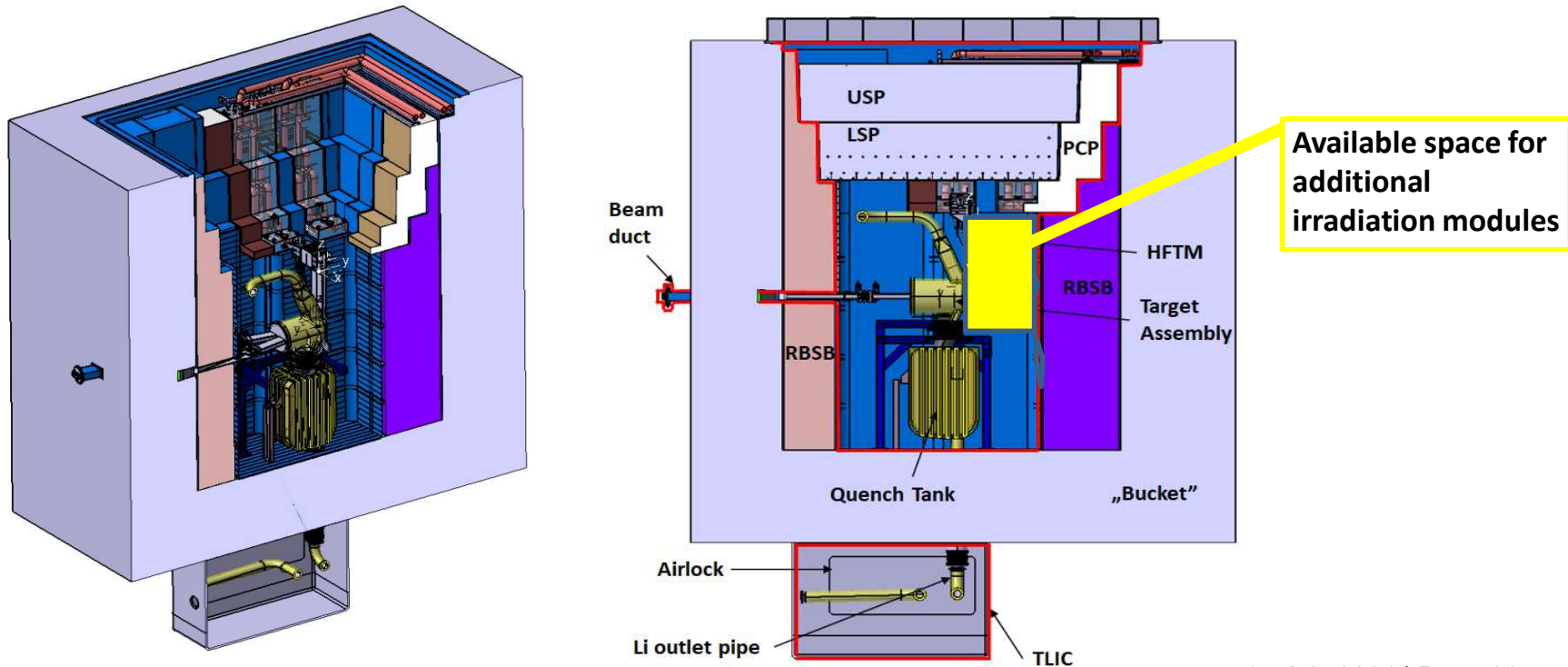
- Up to 800°C, assisted by self-heating
- 8x20 cm³ (cylindrical HT capsules)
- 1–3 dpa/fpy in W
- 9–10 appm He / dpa, (2x of DEMO), 20–29 appm H / fpy, (3x of DEMO)



Adaptation for ODS-steels and vanadium materials can be easily implemented

Other modules ...

Other irradiation modules are also feasible (sequentially or simultaneously to the HFTM)



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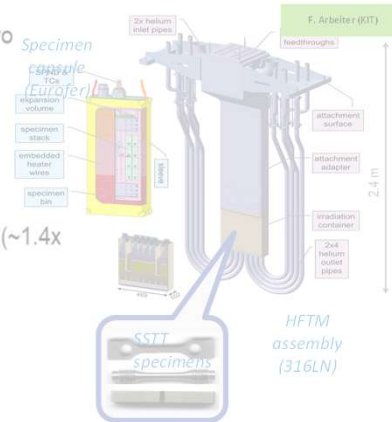
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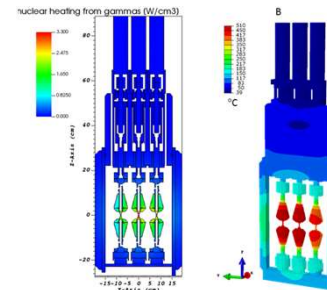
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Adaptation for ODS-steels and vanadium materials can be easily implemented

Prospective irradiation modules for other materials properties characterization are feasible and proposed

• In-Situ Creep Fatigue Test Module (ICFTM)



In-situ creep/fatigue/crack-growth loading & measurement
 Temperature range 250 – 550 °C in the high flux zone
 Base materials, welds, dissimilar welds; optionally multiaxial loads

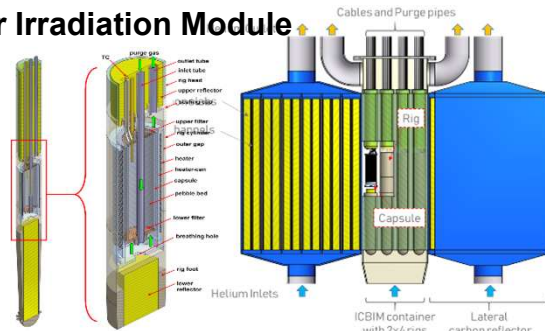
- Breeding Blankets relevant technologies**

The different types of irradiation modules allow to address BB technologies issues which are key pending ones for accelerating fusion as an energy source. The facility design allows the installation of other materials or other irradiation modules (sequentially or simultaneously with the HFTM)

Prospective irradiation modules for tritium technologies validation

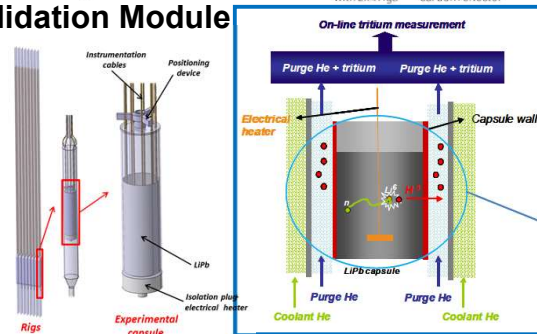
- In-Situ Ceramic Breeder Irradiation Module**

In-situ irradiation and testing of ceramic breeder materials or Be in the temperature range 300 – 1000 °C in the medium flux zone, measuring tritium release



- In-Situ Liquid Breeder Validation Module**

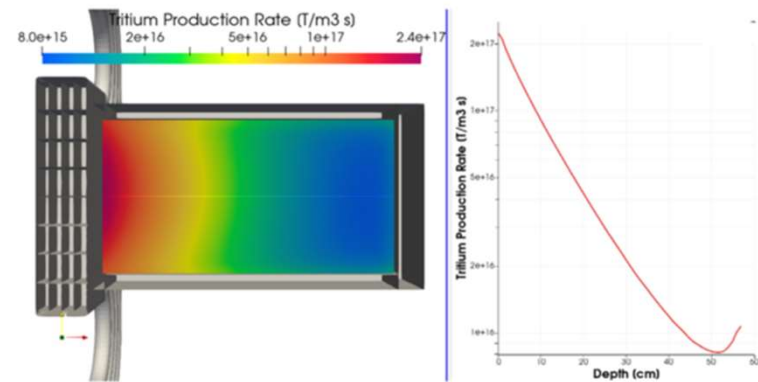
In-situ irradiation and testing of different containers of PbLi in the temperature range 300 – 600 °C in the medium flux zone, measuring tritium release, permeation and extraction techniques



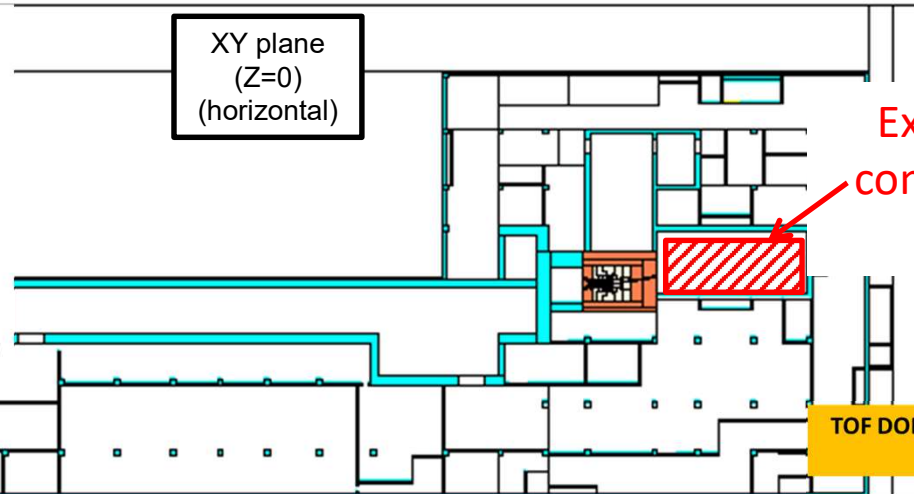
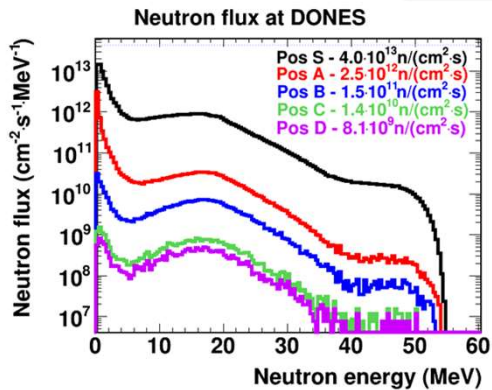
Prospective irradiation modules for functional testing of Model Blanket Module

Interest raised based on:

- An irradiation area similar in size to the typical “unit size” of different BB
- Neutron axial gradient similar to the one in DEMO
- Feasibility of heat loads similar to the one in DEMO first wall

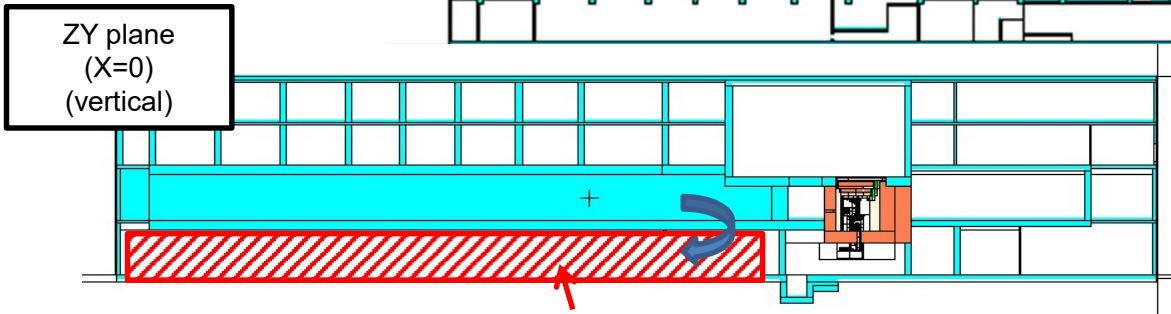


Very high neutron flux at very high energies!!!



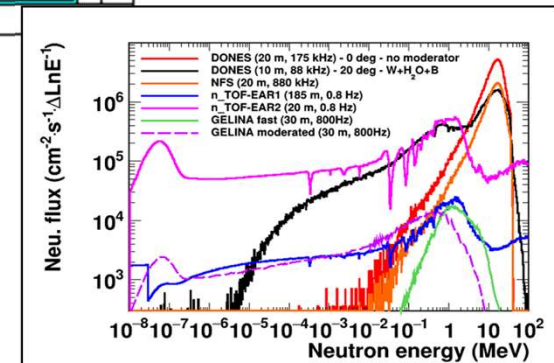
Experiments with continuous neutron beams

TOF DONES would be world's highest intensity TOF neutron source



Experiments with pulsed deuteron/neutron beam

Magnets would deflect the beam downwards

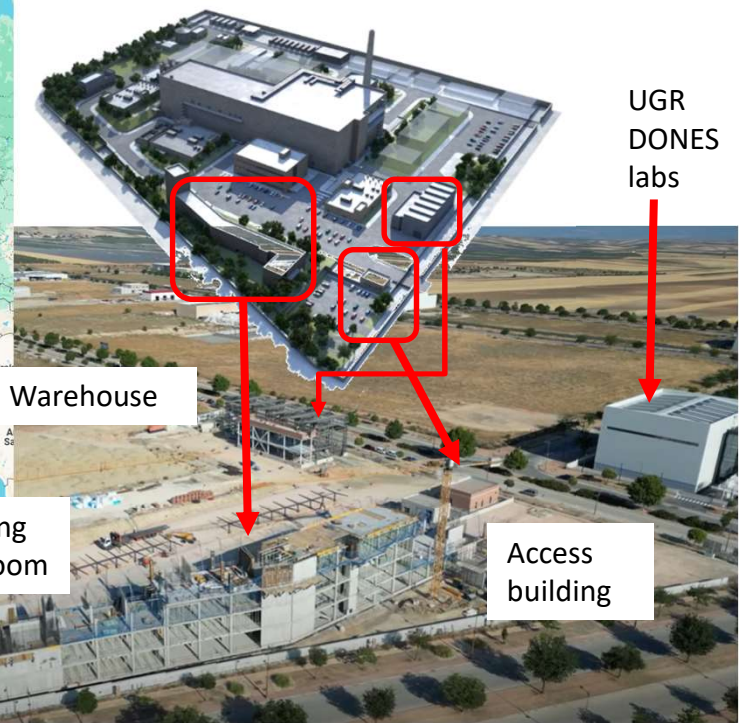
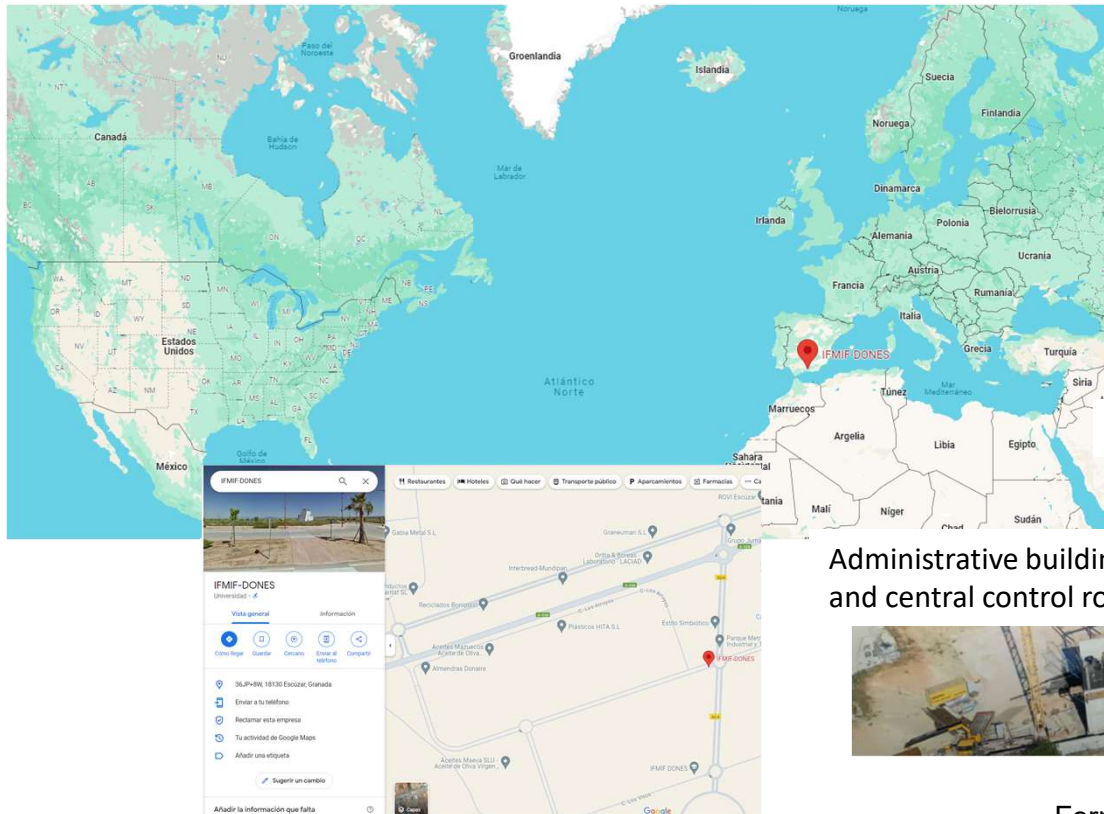


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The Site

It is located in the Granada province (Andalusia region – southern Spain), 18 km southwest from Granada city in the Granada Metropolitan park (Escúzar)

[IFMIF DONES Work Progress: June 2024 \(youtube.com\)](https://www.youtube.com/watch?v=...)



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- **Materials**, with emphasis in **irradiation effects**, is one of the key pending issues in the development of fusion as an energy source
- IFMIF-DONES is the **EU proposed fusion-like neutron source** being built in Granada (Spain)
- IFMIF-DONES is based on a high current D accelerator hitting on a liquid Li moving at high speed. It will allow irradiation of around 1000 engineering-relevant samples at a dose rate higher than 10 dpa/fpy. The **engineering design** of the facility has been **developed** during the last 7 years
- Facility design is **flexible** enough to accommodate different irradiation needs that will evolve along the time
- The Project is progressing properly gaining momentum, **international consensus** and technical readiness.



Thank you for your attention



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