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Fusion R&D on the way towards fusion energy



EUROfusion

Robert Wolf, Wolfgang Biel, Robert Stieglitz, Felix Warmer and the Wendelstein 7-X Team



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R. WOLF 1

Nuclear Fusion

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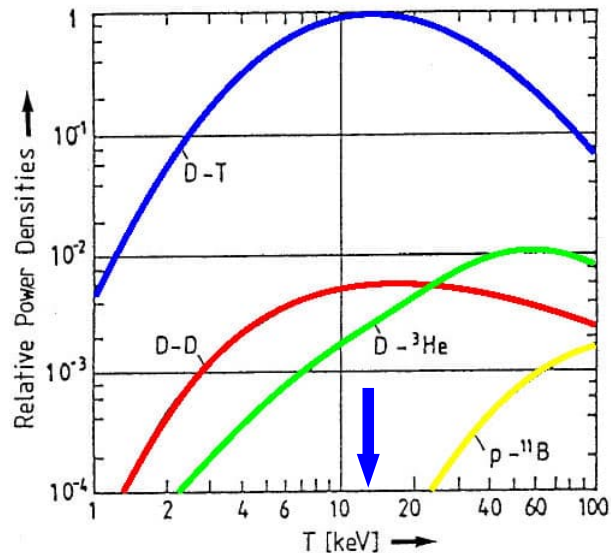
R. WOLF 2

Fusion reactions: Why D-T fusion?

- D-T-fusion has the highest reaction rate of all fusion reactions



- Optimum temperature range of D-T-fusion: 10-20 keV or ~100 M°C
- Too low temperatures: Fusion reactivity rapidly decreases with decreasing temperature
- Too high temperatures: Prohibitive synchrotron losses (in MCF systems)



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Fusion reactions: Why fusion?



$$1 \text{ GW}_{\text{th}} \leftrightarrow 0.3 \text{ kg/Tag}$$



$$1 \text{ GW}_{\text{th}} \leftrightarrow 10^6 \text{ kg/Tag}$$

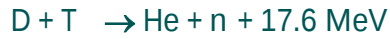
Important aspects

- Neutrons used to breed tritium: Closed fuel cycle
- Fusion “ash” is helium: No CO₂ production
- Neutrons activate material surrounding the plasma
- Avoid long-term waste, recycling after < 100 years

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R. WOLF 4

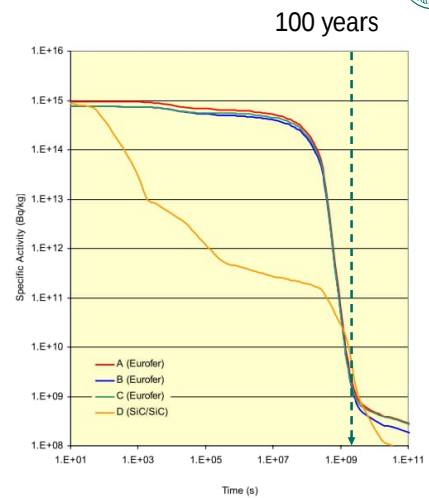
Fusion reactions: Why fusion?



$$1 \text{ GW}_{\text{th}} \leftrightarrow 0.3 \text{ kg/Tag}$$

Important aspects

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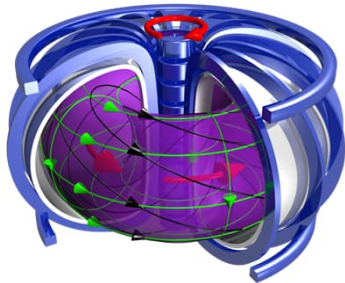
Final Report of the European Fusion
Power Plant Conceptual Study
EFDA(05)-27/4.10

Magnetic confinement fusion (MCF)



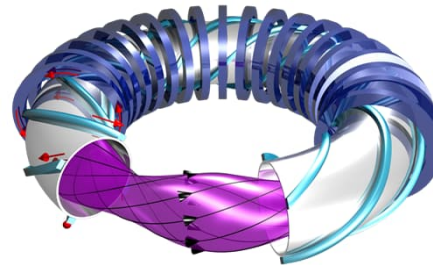
Magnetic confinement concepts

Tokamak



- Simpler geometry
- Usually pulsed operation
- Steady state using current drive
- ITER is a tokamak

Stellarator



- More demanding geometry
- Magnetic field only from external coils
- Intrinsically steady-state
- Wendelstein 7-X is a stellarator

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Magnetic confinement fusion (MCF) parameters

Deuterium-Tritium-Plasma

10^{20} m^{-3}

100 Million °C (100 MK)

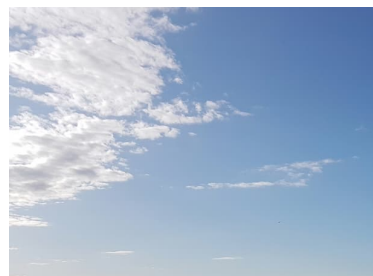
~5 bar

Luft

10^{25} m^{-3}

20 °C (300 K)

1 bar



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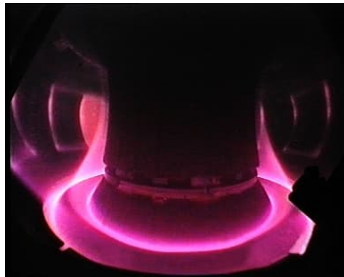
Magnetic confinement fusion (MCF) parameters

Deuterium-Tritium-Plasma

10^{20} m^{-3}

100 Million °C (100 MK)

~5 bar



Plasma state due to high temperatures

Magnetic confinement due to Lorentz force acting on charges plasma particles

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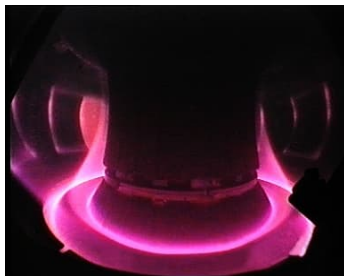
Inertial confinement fusion (ICF) parameters

Deuterium-Tritium-Plasma

10^{20} m^{-3}

100 Million °C (100 MK)

~5 bar



Deuterium-Tritium-Plasma

10^{32} m^{-3}

10 Million °C (10 MK)

~100 Gbar

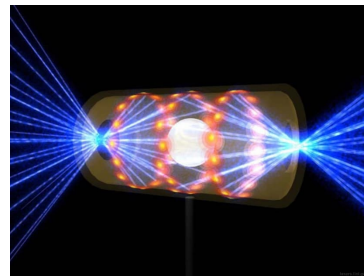


Foto: Lawrence Livermore National Laboratory

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Inertial confinement fusion (ICF) parameters



Deuterium-Tritium-Plasma

10^{32} m^{-3}

10 Million °C (10 MK)

~100 Gbar

Plasma parameters and plasma behaviour fundamentally different in MCF and ICF devices

MCF: Steady-state burning plasma

ICF: Pellet ignition lasting nanoseconds at several Hertz

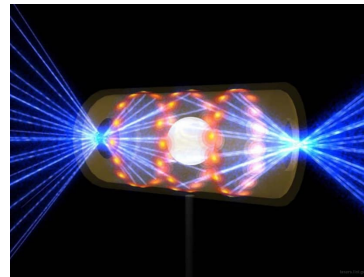


Foto: Lawrence Livermore National Laboratory

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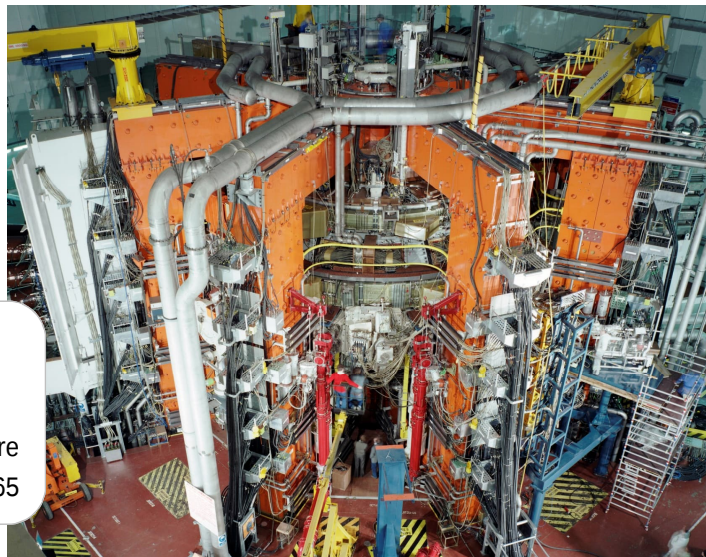
R. WOLF 11

To date largest MCF device: Joint European Torus (JET)



https://de.wikipedia.org/wiki/Joint_European_Torus

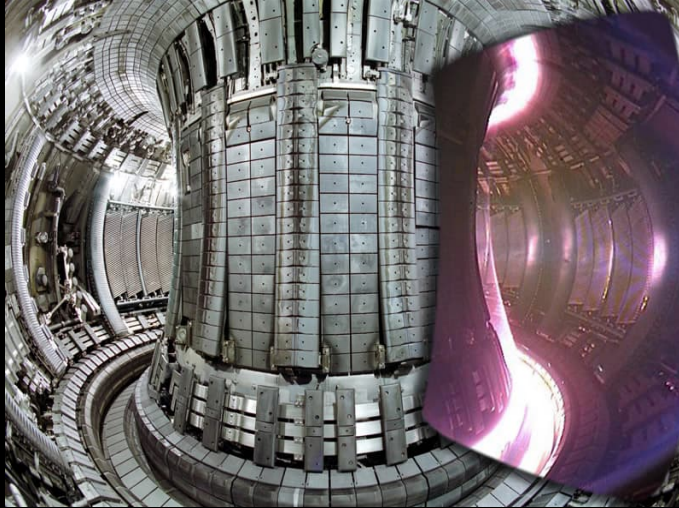
Plasma volume ~ 100 m³
Magnetic field ~3T (copper coils)
One of two MCF experiments worldwide which used D-T-mixture
16 MW of fusion power at $Q = 0.65$



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Joint European Torus (JET)

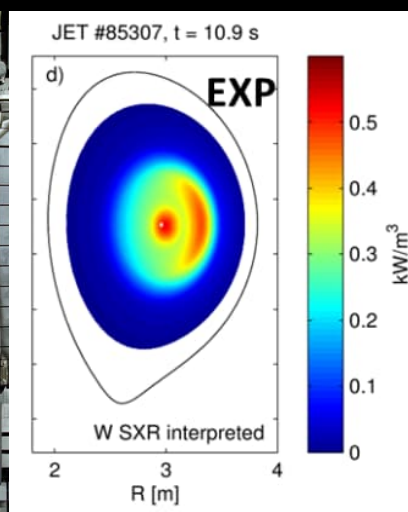
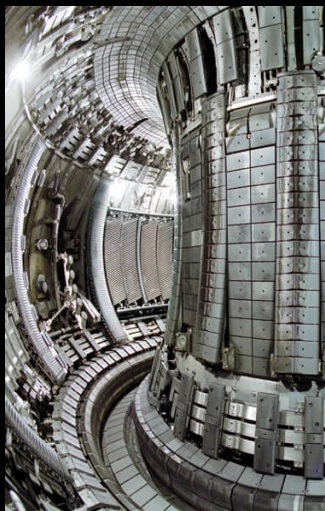


Visible emission
T ~ 100 Mio °C

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Joint European Torus (JET)



X-ray emission
T ~ 100 Mio °C

P. Mantica et al 2015 Proc 42nd EPS
Conferenz on Plasma Physics P1.101

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ITER



Copyright: ITER Organization / E.J.F. Riche

Plasma volume ~ 800 m³

Magnetic field ~6T (superconducting coils)

Burning fusion plasma for the first time

500 MW fusion power for ~10 minutes from
50 MW of heating power ($Q = 10$)

www.iter.org/album/Media/4 - Aerial - Mai 2021

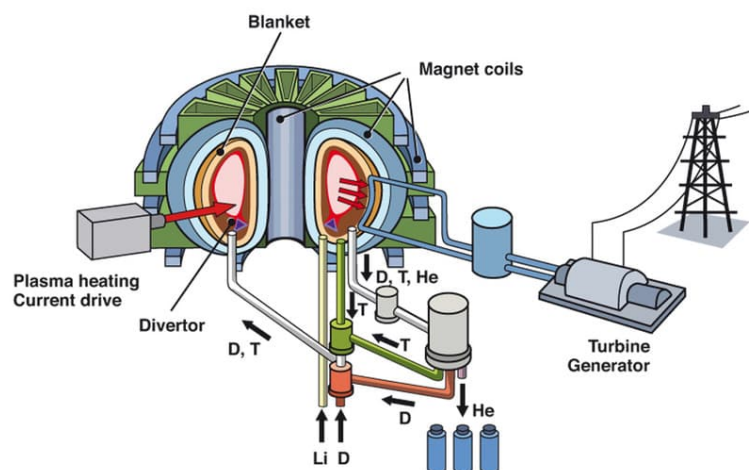
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Layout of a fusion power plant: Integration of physics and technology solutions



- Fusion plasma
- Heat and particle exhaust
- Breeding blanket
- Superconducting magnetic field coils
- Heating & current drive systems
- Fuel cycle



<https://www.iter.org/newsline/255/1481>

Final Report of the European Fusion Power Plant Conceptual Study EFDA(05)-27/4.10

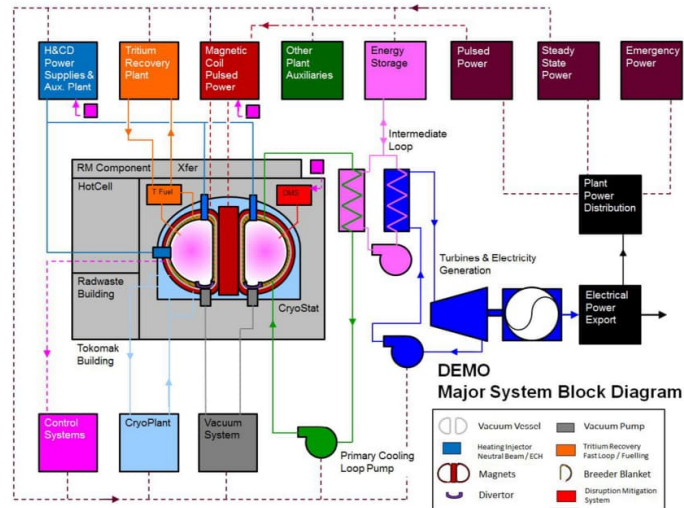
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R. WOLF 16

Layout of a fusion power plant: Integration of physics and technology solutions



- Fusion plasma
- Heat and particle exhaust
- Breeding blanket
- Superconducting magnetic field coils
- Heating & current drive systems
- Fuel cycle



Source ???

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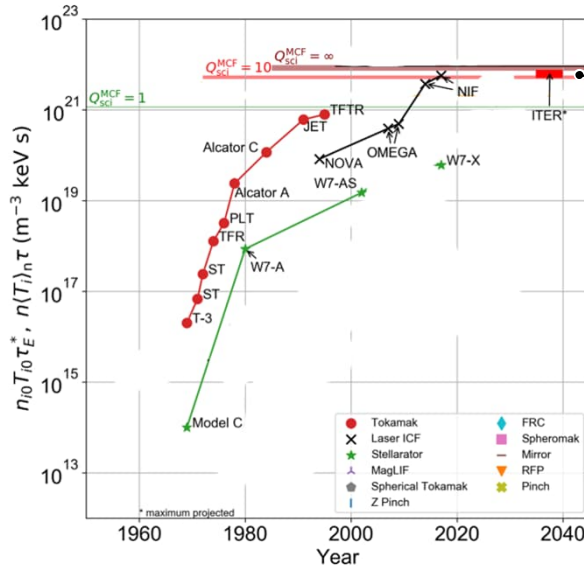
R. WOLF 17

Achievements – where are we?

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R. WOLF 18

Triple product describes the distance to burning fusion plasma or ignition



Burning fusion plasma

$$Q = \frac{P_{fusion}}{P_{heating}} = \frac{5P_{\alpha}}{P_{heating}} \gg 1$$

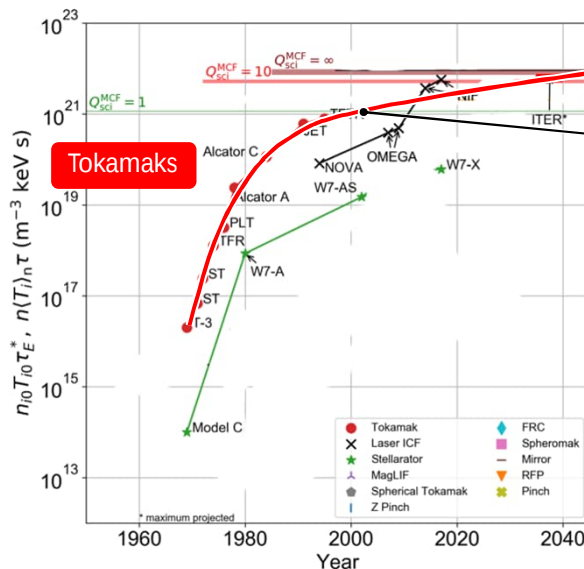
Demonstration of burning fusion plasma only possible at power plant level

S.E.Wurzel, S.C.Hsu, Phys. Plasmas 29 (2022) 062103
doi: 10.1063/5.0083990

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Triple product describes the distance to burning fusion plasma or ignition



Improvement of confinement (heat insulation)
Increase of magnetic field
Increase of size (volume to surface ratio)

Plasma duration from milliseconds to minutes and hours
Transition from copper to superconducting magnetic field coils
Heat exhaust becomes increasingly important

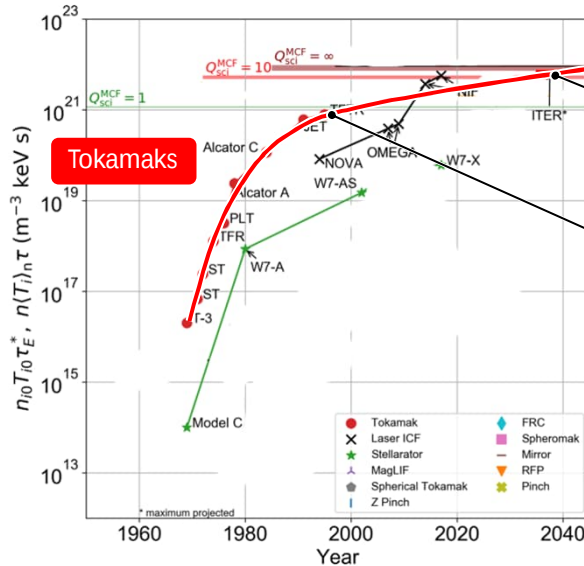
Overall, increasing level of scientific and technical integration and complexity

S.E.Wurzel, S.C.Hsu, Phys. Plasmas 29 (2022) 062103
doi: 10.1063/5.0083990

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Triple product describes the distance to burning fusion plasma or ignition



Energy confinement time has to be increased by an order of magnitude

Essential parameters achieved

- Density $\sim 10^{20} \text{ m}^{-3}$
- Ion temperature $\sim 10 \text{ keV}$

S.E.Wurzel, S.C.Hsu, Phys. Plasmas 29 (2022) 062103
doi: 10.1063/5.0083990

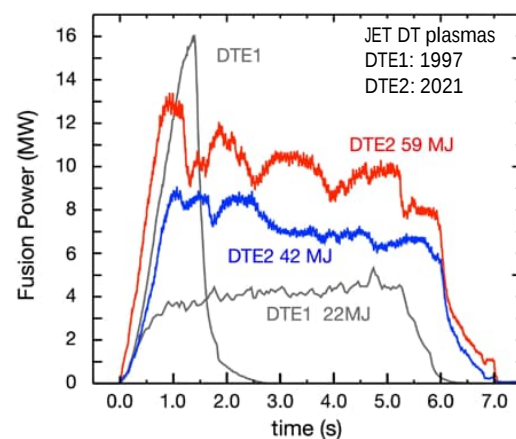
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D-T-fusion energy record in JET



- JET is one of two MCF devices which used tritium
- Physics and technology platform to contribute to the ITER physics and technology base
- DTE1: α -heating with $Q = 0.65$ confirmed ($Q = 1$ corresponds to $\sim 20\%$ of α -heating power)
- DTE2: Extended phase of $\sim 10 \text{ MW}$ of fusion power by optimizing confinement and stability properties and significantly higher heating power

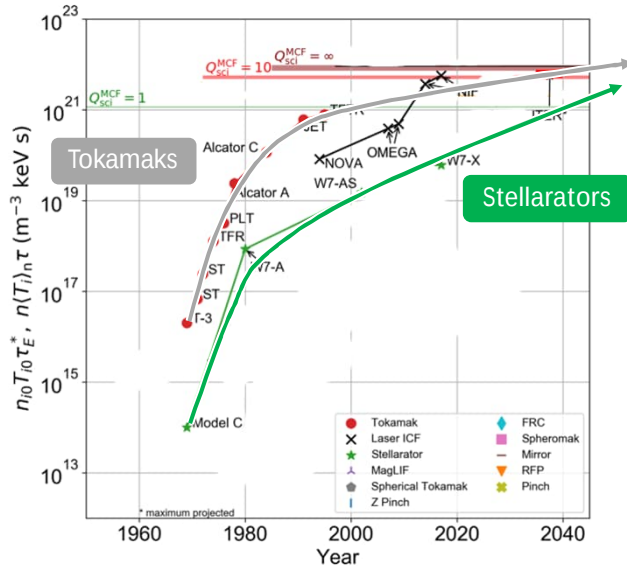


Record fusion energy $\int P_{\text{fusion}} dt$
(from 2021)

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Triple product describes the distance to burning fusion plasma or ignition



Intrinsically steady-state
Soft operational boundaries, much reduced plasma control effort
Overall, optimized stellarator promises more economic power plant

However, requires elaborate physics and technology design optimization to achieve required confinement quality
The design of Wendelstein 7-X was only possible with the advent of super computers

S.E.Wurzel, S.C.Hsu, Phys. Plasmas 29 (2022) 062103
doi: 10.1063/5.0083990

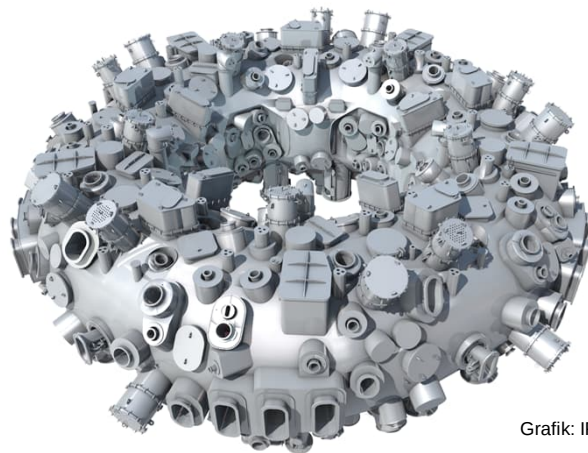
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Wendelstein 7-X is the first fusion device the design of which is based on a comprehensive optimization procedure



Magnetic field
3 T
Superconducting coils
70
Plasma volume
30 m³
Plasma duration
30 minutes
Heating power
10 MW
Peak heat flux
10 MW/m²



Grafik: IPP

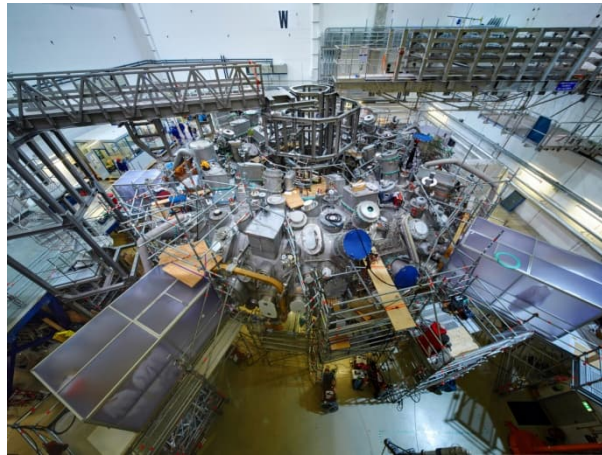
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R. WOLF 24



Wendelstein 7-X is the first fusion device the design of which is based on a comprehensive optimization procedure

Magnetic field
 3 T
 Superconducting coils
 70
 Plasma volume
 30 m³
 Plasma duration
 30 minutes
 Heating power
 10 MW
 Peak heat flux
 10 MW/m²



IPP, Foto: Bernhard Ludewig

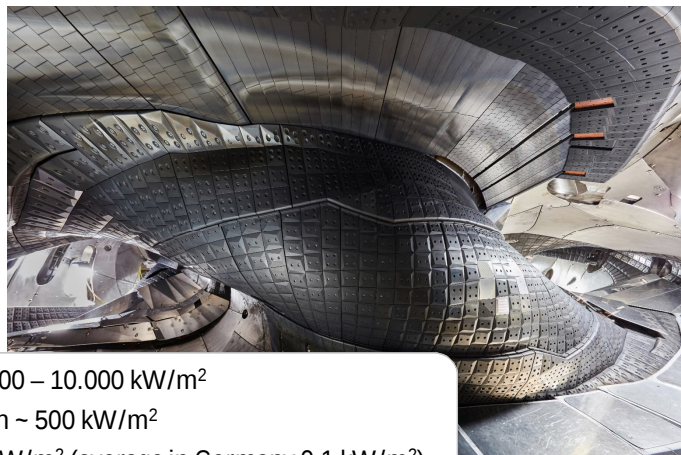
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R. WOLF 25



Wendelstein 7-X is the first fusion device the design of which is based on a comprehensive optimization procedure

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 Plasma duration
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 Heating power
 10 MW
 Peak heat flux
 10 MW/m²



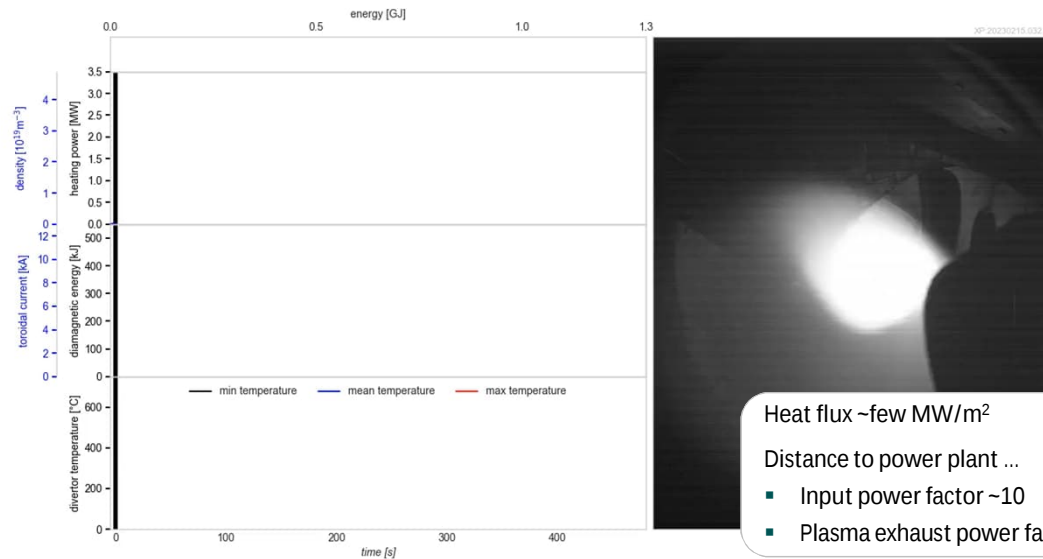
Fusion ~ 1.000 – 10.000 kW/m²
 Fossil, fission ~ 500 kW/m²
 Solar < 1.4 kW/m² (average in Germany 0.1 kW/m²)

IPP, Foto: Bernhard Ludewig

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On the way to a high-power steady-state fusion plasma



Heat flux ~few MW/m²

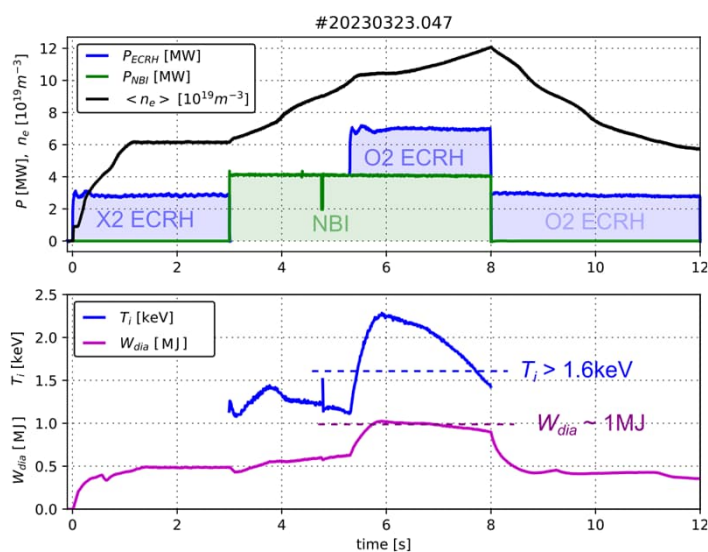
Distance to power plant ...

- Input power factor ~10
- Plasma exhaust power factor ~100

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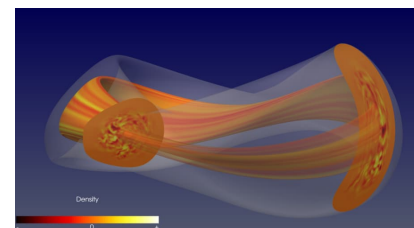
On the way to a high performance plasma



Major optimization criteria verified

- Low plasma currents
- Low collisional transport

However, turbulent losses become important



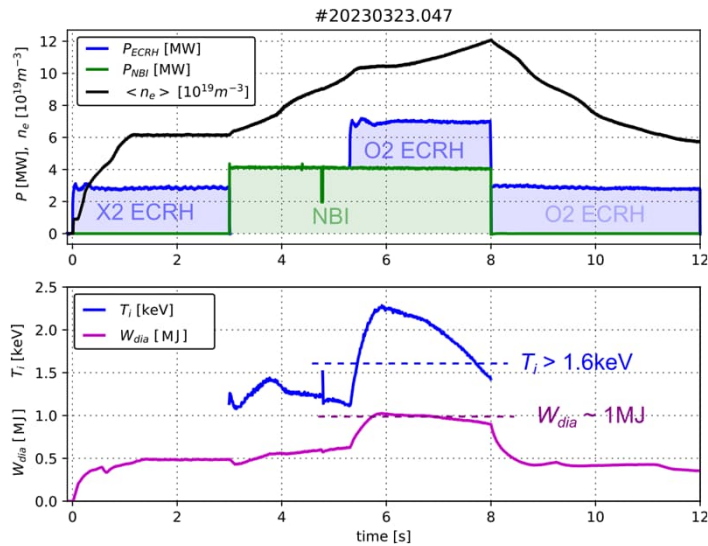
M.Maurer, PhD thesis (2020) TUM
<https://mediatum.ub.tum.de/doc/1539628/1539628.pdf>

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On the way to a high performance plasma



Major optimization criteria verified

- Low plasma currents
- Low collisional transport

However, turbulent losses become important

Distance to power plant ...

- Density achieved
- Temperature factor ~ 5
- Energy confinement ~ 10

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Inertial confinement fusion (ICF)

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National ignition facility (laser driven fusion)



Target chamber

- Commissioning 2009
- Primary objectives are defence applications

Laser installation

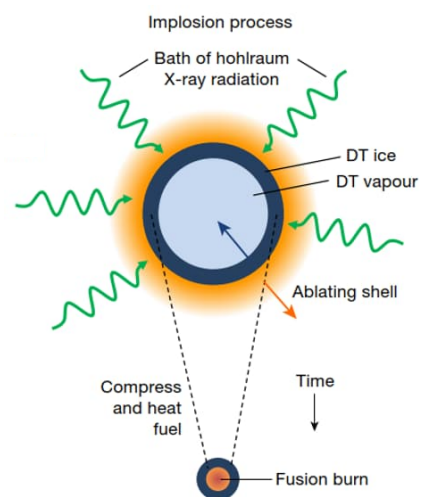
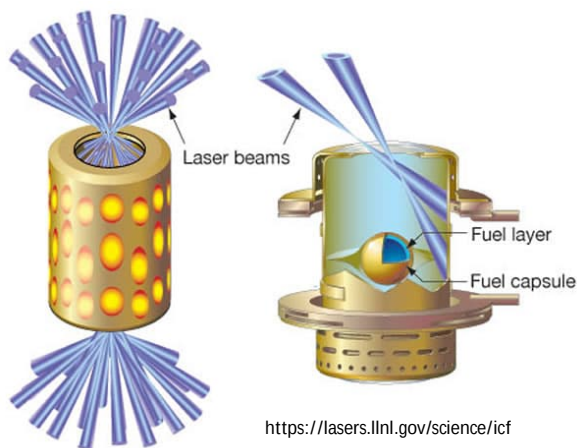


Source:
lasers.llnl.gov/multimedia/photo_gallery/

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Inertial confinement fusion (ICF) principle – indirect drive



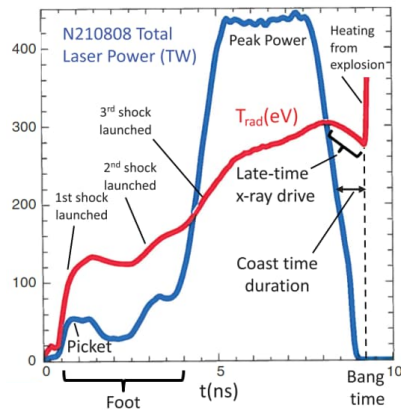
A. B. Zylstra et al. Nature 601 (2022) 542

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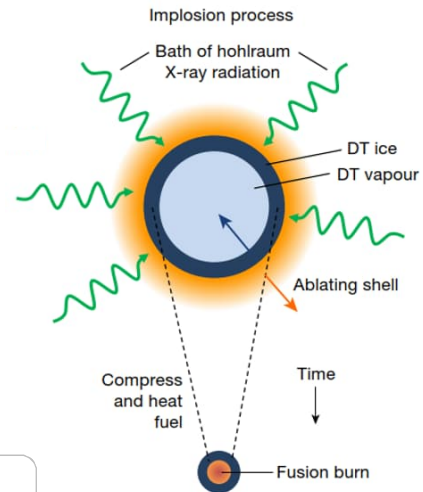
R. WOLF 32

Inertial confinement fusion (ICF) principle – indirect drive

H. Abu-Shawareb et al. Phys. Rev. Lett. 129 (2022) 075001



Ignition Dec '23: 400 MJ input to lasers → 2 MJ laser energy to target → conversion o X-rays 0.2 MJ → 3,1 MJ fusion energy

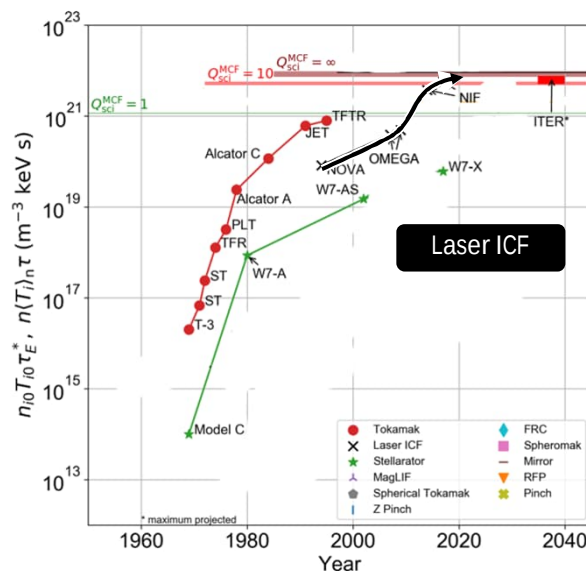


A. B. Zylstra et al. Nature 601 (2022) 542

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Triple product describes the distance to burning fusion plasma or ignition



Important milestone: Demonstration of ignition of DT pellet

Laser ICF: Only conceptual ideas for the realisation of a power plant

- Direct drive
- High frequency (several Hertz)
- Heat exhaust

MCF: Relevant technologies were from the beginning part of the development

S.E.Wurzel, S.C.Hsu, Phys. Plasmas 29 (2022) 062103
doi: 10.1063/5.0083990

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Prospects – the way forward

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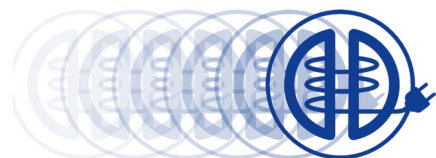
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Key elements for the realization of fusion energy

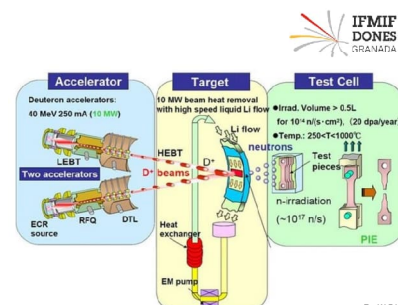
- ITER: First demonstration of a burning fusion plasma & first time breeding tritium from fusion neutrons and lithium
- Wendelstein 7-X: Validation of an alternative confinement concept with favourable power plant properties
- Development of the first integrated design of a demonstration power plant (DEMO)
- 14 MeV neutron source (IFMIF-DONES) for the material qualification for DEMO



European Research Roadmap
to the Realisation of Fusion Energy



www.euro-fusion.org/eurofusion/roadmap/

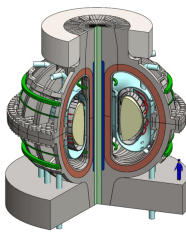


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New developments

- Emerging technologies
 - High-temperature superconductors
 - Composite materials
 - Additive manufacturing
- Discussion about regulatory framework
- 4.7 b\$ worldwide in fusion start-ups



Affordable, robust, compact (ARC) reactor

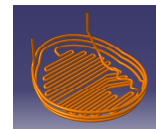
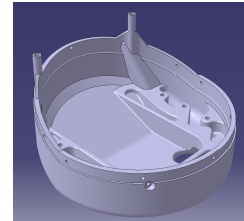
B.N. Sorbom et al
Fusion Engineering and Design 100 (2015) 378

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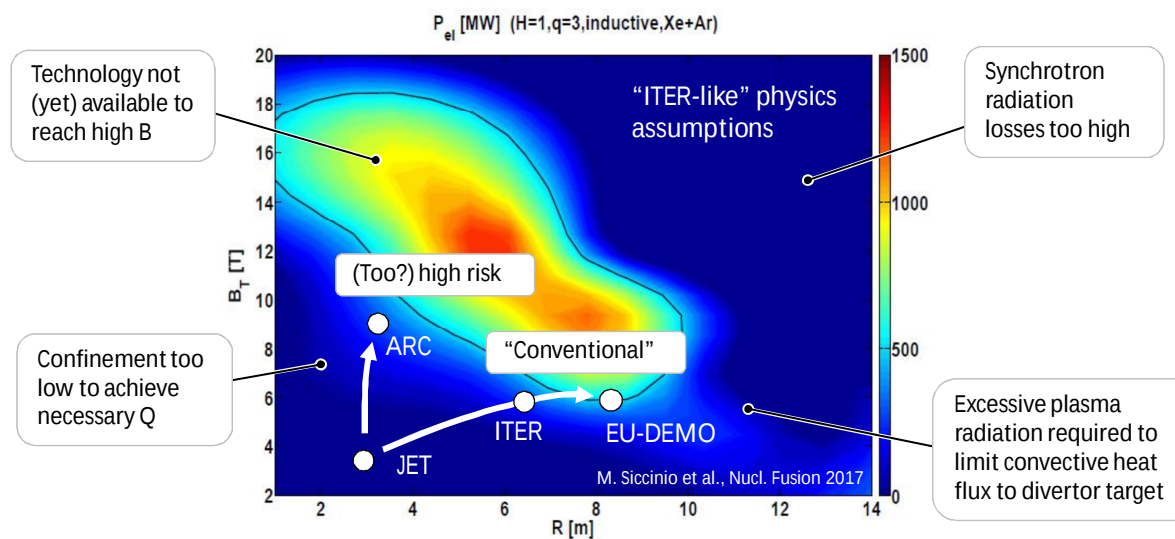
Foto: ITEP, KIT

3D printed steel structures
Source: IPP



R. WOLF 37

Design space boundaries

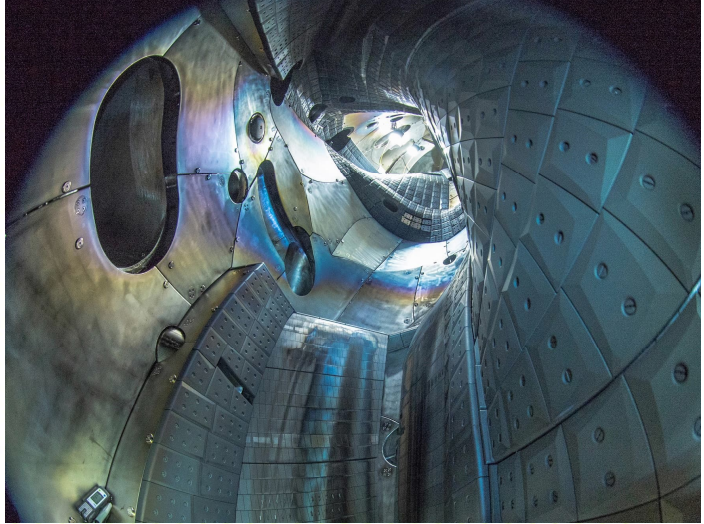


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Thank you for your attention

View into the plasma vessel of Wendelstein 7-X
Courtesy C. Biedermann, G. Wurden



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