

# Comparative assessment of different approaches for the use of CAD geometry in Monte Carlo transport calculations

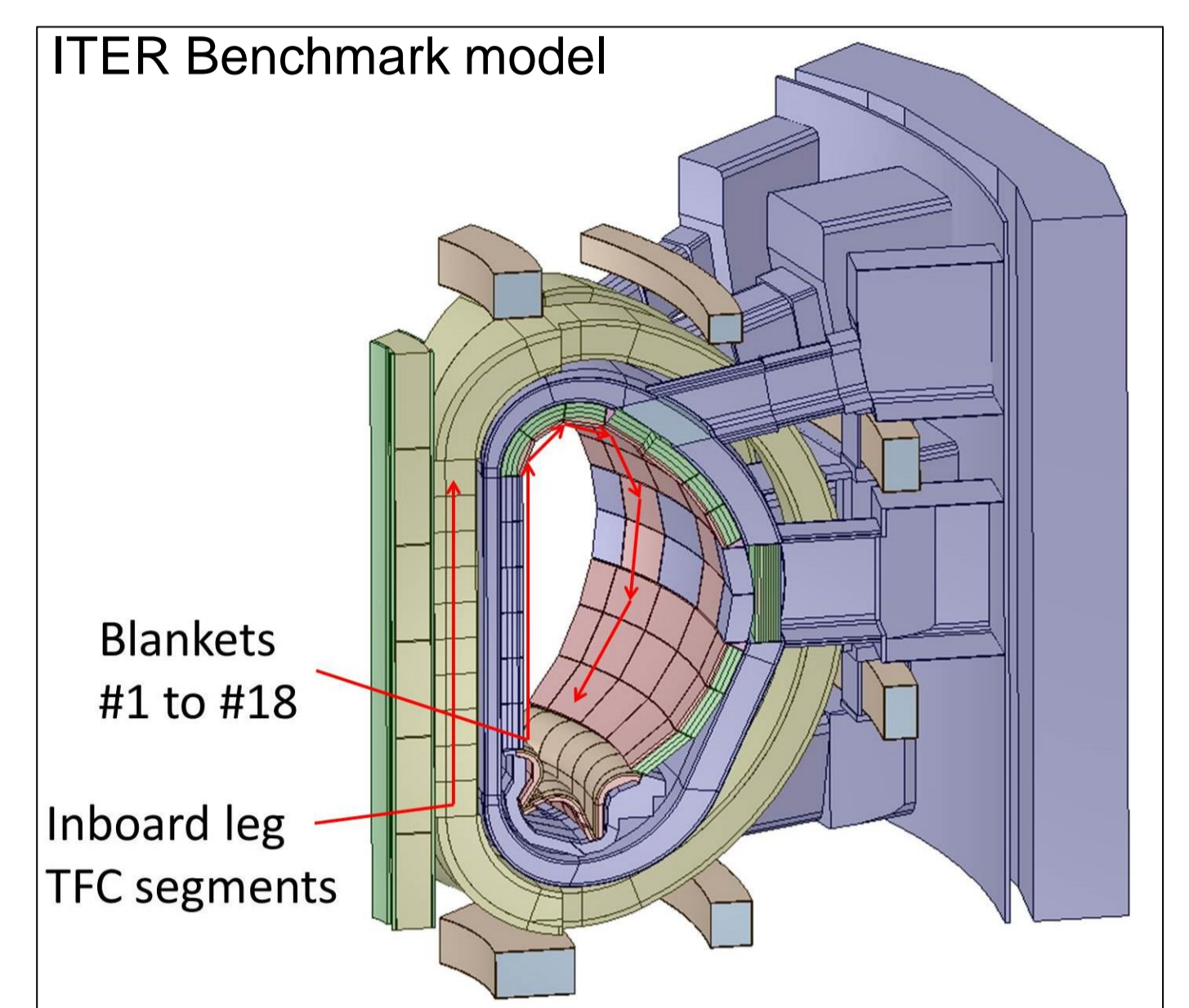
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## Objectives

- Single (unique) CAD geometry: ITER CAD benchmark model.
- Three different approaches for use of CAD geometry in MC transport calculations:
  - McCad:** Current standard approach, conversion of CAD geometry into MCNP representation using McCad developed at KIT.
  - MCNP6&TT:** Using MCNP6's unstructured mesh geometry feature, meshing of CAD geometry with the tessellation-tetrahedralization (TT) approach developed at KIT.
  - DAGMC:** Direct particle tracking on the CAD geometry using a patched version of MCNP developed at UW-Madison.
- Comparison with respect to performance and user-friendliness:
  - Installation** (Installation guide, needed software, installation complexity).
  - Model preparation** (repairing geometry error, time needed, user expertise).
  - Computation performance** (calculation speed, accuracy).



## Installation



- Good installation guide.
- Only one additional software package required.
- Open Source software (except MCNP).
- Simple installation.



## MCNP6&TT

- New development, up to now no installation guide.
- several software packages needed but not interdependent.
- Open Source Software (except MCNP).
- Moderate installation



## DAGMC

- Installation guide available, but not sufficient.
- Interdependent software packages. Dependent on specific, older versions. Cubit and MCNP under license control.
- Complex installation.

## Model preparation



- Decomposition into convertible solids. Substitution of spline surface with analytical surfaces mandatory.
- Iterative and time intensive.
- Extensive user expertise required.



- Removing overlaps/gaps of solids.
- Fast conversion.
- User expertise required for optimization of meshing.
- Tally definition difficult due to meshing of cells and surfaces.



- No user guide for repairing geometrical errors.
- Moderate speed, iterative steps required.
- User expertise essential.
- Tally definition very convenient, more user-friendly than standard MCNP.

## Computation performance



- Fastest calculation.
- Current standard approach for MCNP calculations, chosen as reference.



- Slowest calculation; speed depends mainly on mesh resolution.
- Superimposed mesh gives large deviation for deep penetration calculations.
- Cell tallys agree with McCad results within statistical errors.



- Moderate calculation speed.
- If model preparation done correctly, very good agreement with McCad results.
- First wall cell tallys within 1% of McCad results.

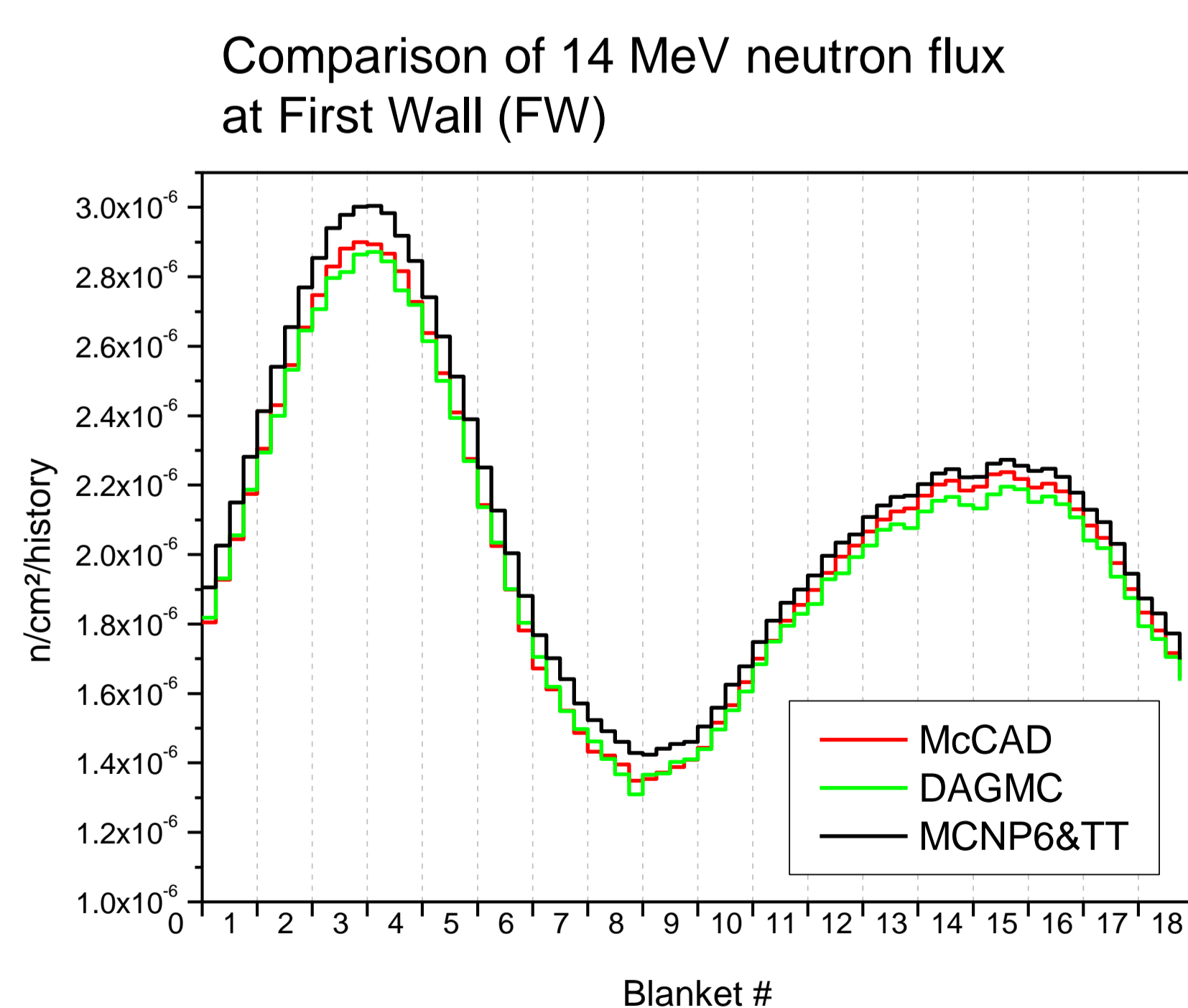
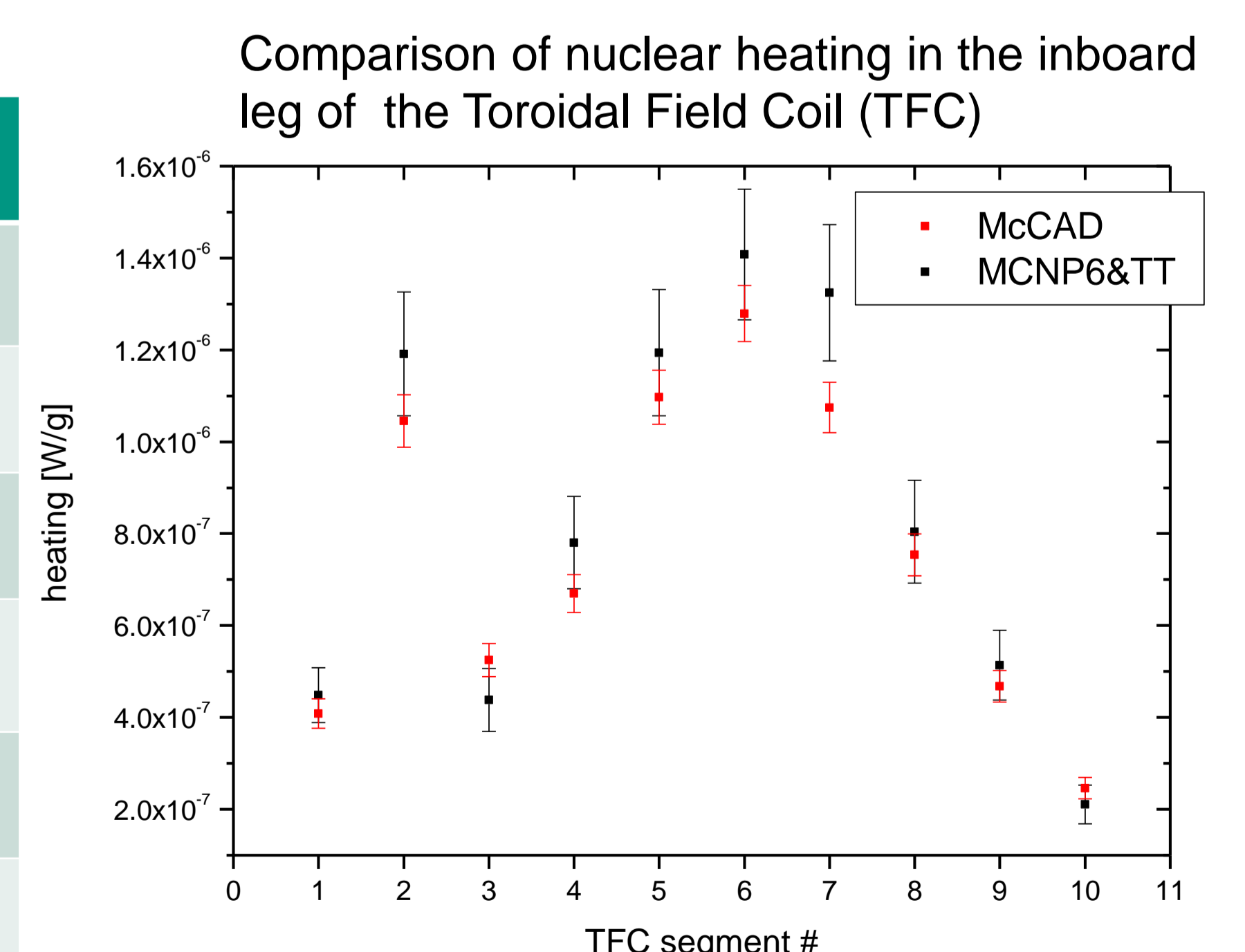


Table: Comparison of calculation performances.

	McCAD	MCNP6 & TT	DAGMC
Lost particles	4e <sup>-6</sup> %	2e <sup>-4</sup> %	3e <sup>-4</sup> %
CPU time per 1e5 histories (voided)	23 s	550 s	134 s
CPU time per 1e5 histories	294 s	1004 s	630 s
Deviation from McCad FW 14 MeV neutron current	-	3.4 %*	1.0 %*
Deviation from McCad heating inboard TFC	-	9.8 %*	too many lost particles
Deviation from McCad Neutron flux mesh at equatorial port	-	27 %	too many lost particles

\*within statistical error



## Conclusions

- McCad most useful for simple models or if model preparation has to be done only once. Small changes to the geometry can be done directly in the MCNP input file.
- MCNP6&TT model preparation extremely fast and reliable. Problematic with regard to nuclear responses; for meshtally the deviation to McCad approach larger than statistical error. Statistical error in general larger than for McCad approach (same number of histories).
- DAGMC most useful for complex models that need to be changed regularly.