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Theoretical and Computer Analysis on the Behaviour of Fast Reactor Fuel Pins and Related Parts of the Core under Operational Conditions

> G. Karsten, H. Kämpf, A. Gerken with Contribution of M. Guyette



GESELLSCHAFT FUR KERNFORSCHUNG M.B.H.

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Gesellschaft für Kernforschung mbH, Karlsruhe

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1. Introduction

The maximum operation times of the fuel pins of a power reactor are of especially great importance for the economy of the reactor core. Therefore a great number of burnup irradiation experiments are performed. Theoretical efforts have been done to optimize the design of irradiation experiments. This was supported by an evaluation of numerous reports on irradiation and basic experiments. Thus a development of a theoretical basis of fuel pin operation behaviour and correspondingly a theoretically founded, generally valid, fuel pin design was started.

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In this work a study of the maximum operation times of the fuel pins was made by following the reference design study on the Sodium Cooled Fast Breeder Reactor Na-2. As the basis of this examination there was taken a defined three zone volume balance relating to the swelling of fuel volume and the consumption of void volume, given by the hot gap volume, the axial expansion and ratios of the pore volume. To calculate the maximum burnup with this model the radial temperature distribution had to be treated quantitatively. For doing this a new porosity relationship of the thermal conductivity was used, which was developed recently. Furthermore a two zone porosity model was applied for the characterization of the porosity migration in the plastic region connected with the central channel formation and a densification in the plastic region. The used zone swelling rates and applied availability factors were argued. The still unsolved fuel clad interaction problem was characterized and discussed. To initiate the quantitative treatment of the interaction problem a multiaxial stress and strain evaluation of the creeping sheath was carried out by an iterative method.

2. Theoretical assumptions

2.1. Formulation of the burnup model

The basis of this study is the "burnup formula" (1,2) in the following modified form

$$A_{\max} = \frac{P_{o}(G_{p1} \cdot V_{p1} + G_{cr} \cdot V_{cr} + G_{1t} \cdot V_{1t}) + V_{hg} + V_{ax}}{(1 - P_{o})(S_{p1} \cdot V_{p1} + S_{cr} \cdot V_{cr} + S_{1t} \cdot V_{1t})}$$
(1)

This formula is basing on the following model: The space within the clad of a fuel pin is divided into three annular temperature zones, the "plastic zone" for temperatures above $T_{p1} = 1700^{\circ}C$, the "creep zone" for temperatures between $T_{p1} = 1700^{\circ}C$ and $T_{cr} = 1300^{\circ}C$ and the "low temperature zone" for temperatures below $T_{cr} = 1300^{\circ}C$. The maximum burnup A_{max} is reached, if in an axial volume element of the space within the clad the fuel volume produced in the three zones according to specific constant swelling rates S_{p1} , S_{cr} and S_{1t} has filled the given void volume without interaction with the cladding. This is valid under provision of a free displaceability of the fuel. It is assumed furthermore, that the void volume is realized on the one hand by the original porous volume, which is given by the porosity P_o and which is available in the three fuel zones according to specific constant availability factors G_{p1} , G_{cr} and G_{1t} . On the other hand the swelling fuel can fill the hot gap volume V_{hg} and can partially expand in axial direction with the volume V_{av} .

$$A = burnup \frac{MWd}{t(heavy elements)}$$

$$S = \frac{1}{A} \frac{\Delta V}{V_{th}} = swelling rate \frac{v/o (theoretical dense)}{10^4 \frac{MWd}{t(heavy elements)}} = 10^{-6} \frac{t}{MWd}$$

$$V_{th} = volume of the theoretical dense fuel$$

$$V_{i} = volumes$$

$$G_{i} = availability factors \int i = pl, cr, lt$$

$$P_{o} = initial, (as-fabricated) porosity$$

The maximum operation time t_{max} is combined with the maximum burnup A_{max} in the following way

$$t_{max} = \frac{\pi \rho_h r_f^2 (1 - P_o)}{\chi} \cdot A_{max}$$
(2)

t = maximum operation time

 ρ_h = density of the heavy elements of the theoretical dense fuel

r_f = fuel radius

 χ = linear rod power

Considering (1) and (2) we can see that the maximum operation time varies within the core with respect to the linear rod power χ , the temperature volumes V_{p1} , V_{cr} , V_{1t} and V_{hg} which are given by the temperature distribution. This is calculated in chapter 2.2. Furthermore the maximum operation time is given by the swelling rates, the availability factors and the axial volume expansion. These values are discussed in chapter .2,3. and 2.4.

2.2. <u>Calculation of the temperature volumes and of the hot gap volume</u>

2.2.1. Porosity dependence of the thermal conductivity

The following relation for isotropic porosity distribution was developed on the base of a mixing model $\langle 3 \rangle$

$$k(T,P) = k_{th}(T) \left[1 - \frac{2}{P^3} \left(1 - \frac{1}{1 + P^3} \left(\frac{k_{th}}{k_p} - 1 \right) \right) \right]$$
(3)

k = thermal conductivity of the porous ceramic fuel
k_{th} = thermal conductivity of the theoretical dense fuel
k_p = thermal conductivity of the pores
T = temperature
P = porosity

The heat transfer in the pore is caused by heat conductivity and temperature radiation. In $\langle 3 \rangle$ the following formula was derived:

$$k_{p} = C_{cond} \sqrt{T_{p}} + C_{r} T_{p}^{3} d \beta$$
 (4)

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C	*	material property for thermal conduction
° _r	=	material property for temperature radiation
d	=	largest length of the pore in direction of the heat flow
ß	=	shape factor
т _Р	=	temperature of the pore

From (3) and (4) results the following approximation

$$k(T,P) = k_{th}(T) \left[1 - P^{\frac{2}{3}} \right]$$
 (5)

which is valid for all practically interesting temperatures and gas contents of the pores and for sufficient small pore sizes, e.g. for Xenon (5) is valid for pore sizes smaller than 40 μ .

(5) is represented graphically in fig. 1. If the curve (5) is replaced by Loebs' straight lines varying rises in succeeding porosity sections are obtained.

$$k(T,P) = k_{th}(T) \left[1 - \gamma P \right]$$
(6)

n = constant relating to linear porosity relation

For	0 < P < 0,1	η	= 2,	5
	0,1 <p<0,15< td=""><td>η</td><td>= 2</td><td></td></p<0,15<>	η	= 2	
	0,15<₽ <0,25	η	= 1,	7

We realize that the thermal conductivity rises more than proportionally with decreasing porosity. These results theoretically obtained are in a very good agreement with experimentally found results $\langle 4$ to 10 \rangle , especially the big value of the constant n at small porosities. Contrary to these results Biancheria $\langle 11, 12 \rangle$ theoretically obtained an opposite direction of the factors i.e. a small value n for small porosities. Biancheria's results based on a publication $\langle 13 \rangle$ about the "Mathematical treatment of the electric conductivity and capacity of disperse systems". There is either an error in this work or the translation of this electric case into the thermal case is impossible.

2.2.2. Thermal dependence of the thermal conductivity

To get the temperature dependence of the thermal conductivity of the theoretical dense fuel $k_{th}(T)$ the results of the following authors (14 to 20) are averaged. These results are relating to stoichiometric UO_2 respectively $(U_{0,8}Pu_{0,2})O_2$ with 95 % of theoretical density. The corresponding values of the theoretical dense fuel follow from the porosity relation (5). These were adjusted according to the method of least squares with the equation

$$k_{\rm th} = \frac{\kappa_1}{\kappa_2 + T} + \kappa_3 T^3$$
 (7)

given by solid state physics. Hence follows < 21 >

$$k_{th}(T) = \frac{41.2}{T+4.9} + 6.55 \cdot 10^{-13} T^{3} + 7.7 (\frac{W}{cm^{\circ}C})$$
(8)
T in °K 900°K $\leq T \leq 3000^{\circ} K$

On the base of the deviations of the several authors' results an error of k_{th} of $\stackrel{+}{-}$ 7 % can be considered as realistic.

In fig. 2 the thermal conductivity is represented graphically as a function of temperature with the porosity as parameter. The decreasing drop of the thermal conductivity with growing porosity can be seen.

2.2.3. Calculation of the temperature volumes

According to the burnup model formulated in chapter 2.1. there are three temperature zones in the fuel, the volumes of which are necessary for the calculation of the maximum burnup. The thermal consequences of the porosity migration towards the fuel axis in the plastic region that is for temperatures above $T_{pl} = 1700^{\circ}$ C-is taken into account. This porosity migration creates a central channel and densifies the plastic fuel. This effect is characterized by a two zone porosity model < 21, 22>. Thus the following model is applied:



Fig. 3 : Radial cross section of the fuel model

Indices:

- cc = central channel
- pl = plastic zone
- cr = creep zone
- lt = low temperature zone
- f = fuel (-surface)

Symbols:

r _c =	fuel radius	T _c =	$T(r_{r})$
$r_{ar}^{I} =$	radius of the creep-zone	$T_{ar}^{r} =$	$T(r_{r}^{I}) = 1300^{\circ}C$
$r_{n1}^{CL} =$	radius of the plastic zone	$T_{1}^{C1} =$	$T(r_{n1}^{C1}) = 1700^{\circ}C$
$r_{cc}^{PT} =$	central channel radius	$T^{p_1} = cc$	$T(r_{cc}^{p_1}) = T_{max}$

Derived from the heat balance equation and the Fouriers' law of the proportionality of heat flux density and thermal gradient for radial heat flow the following relations for the ratios of the sectional area result



Substituting (5) and (7) in (9) and (10) the corresponding ratios of the axial volume elements are obtained.

$$\frac{v_{1t}}{v_{f}} = \frac{4\pi}{\chi} (1 - B_{o}^{3}) \left[\varkappa_{1} \ln \left(\frac{\varkappa_{2} + T_{cr}}{\varkappa_{2} + T_{f}} \right) + \frac{\varkappa_{3}}{4} (T_{cr}^{4} - T_{f}^{4}) \right]$$
(11)

$$\frac{V_{cr}}{V_{f}} = \frac{4\pi}{\chi} \left(1 - P_{o}^{3}\right) \left[\varkappa_{1} \ln \left(\frac{\varkappa_{2} + T_{p1}}{\varkappa_{2} + T_{cr}} \right) + \frac{\varkappa_{3}}{4} \left(T_{p1}^{4} - T_{cr}^{4} \right) \right]$$
(12)

$$\frac{v_{p1}}{v_{f}} = 1 - \frac{v_{1t}}{v_{f}} - \frac{v_{cr}}{v_{f}}$$
(13)

A common constant availability factor G_{pl} is used for the densified plastic region and for the central channel. In the region for temperatures above $T_{pl} = 1700^{\circ}C$ there is only a dislocation of the fuel volume so that the volume of the theoretical dense fuel does not change. Therefore we can take the original not densified plastic volume V_{pl} in equation (1), so that the porosity migration has no influence on the volume ratios above.

2.2.4. Calculation of the hot gap volume and the central temperature

From some irradiation experiments the heat transfer coefficient fuel-cladding during operation time was found to be

$$h_{hg} = 1, 1 \frac{W}{cm^{2}c}$$
 (14)

In spite of uncertainties according to this value and its change during operation time this values is taken constant. It follows from an analysis of this value $\langle 2, 23 \rangle$ that there exists a partial contact of the fuel with the cladding so that during operation time a defined gap width is not given. Furthermore it is known from irradiation experiments that the fuel is cracked partially. That means that the original gap volume after having changed by thermal expansion of fuel and cladding is localized in cracks, too. To calculate the maximum burnup according to (1), it is irrelevant where the hot gap volume V_{hg} is localized.

In the following we consider an axial fuel volume element and calculate the increase of volume in radial direction. The porosity is fixed constant. The fuel can be regarded to be free of thermal stresses. In the plastic region the thermal stresses are relaxed, in the brittle region the thermal stresses have caused the cracks. Therefore we can proceed as follows: The initial ring element $2\pi r_0 dr_0$ is put plastically and incompressible around the circle with radius r given by thermal expansion of the region with radius r_0 . There it expands according to the thermal expansion coefficient α from the constant initial temperature T_0 to the temperature T(r).

The following differential equation is the result:

$$2\pi r dr - 2\pi r_o dr_o = 2\pi r_o dr_o^2 \int_{0}^{1(r)} \alpha(T) dT \qquad (15)$$

We integrate (15) over the fuel radius r_f for the following assumptions:

a) $\alpha_{f} = \alpha_{1} T + \alpha_{2}$ whereby $\alpha_{1} = 3,9 \ 10^{-9} \frac{1}{\sigma_{C}^{2}}$ $\alpha_{2} = 7,87 \ 10^{-6} \frac{1}{\sigma_{C}}$ (24)

b) Parabolic temperature distribution which does not change during thermal expansion (that is constant thermal conductivity, constant heat source density and no central channel)

$$T(r) = T(r_0) = T_{cc} - \frac{\chi}{4\pi k} \frac{r^2}{r_f^2}$$

c) $T_{0} = 0^{\circ}C$

The following relative radial volume increase of the axial volume element is obtained

$$\frac{\Delta V_{f}}{V_{f}} = \alpha_{2}(T_{cc} + T_{f}) + \alpha_{1}T_{cc}T_{f} + \frac{\alpha_{1}}{3}(T_{cc} - T_{f})^{2}$$
(16)

For the relative hot gap volume results at last:

$$\frac{V_{hg}}{V_f} = 2\frac{\delta}{r_f} - \alpha_2(T_{cc}+T_f) - \alpha_1T_{cc}T_f - \frac{\alpha_1}{3}(T_{cc}-T_f)^2 + 2\alpha_{sh}T_{sh}$$
(17)

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 \mathcal{A}_{sh} = thermal expansion coefficient of cladding \mathcal{T}_{sh} = radial averaged cladding temperature

The fuel surface temperature T_f is essentially given by the heat transfer coefficient (14).

For the calculation of the fuel central temperature two cases must be distinguished:

a) $T_{cc} \stackrel{\leq}{=} T_{p1}$

that means that we get a solid cylinder with homogeneous porosity distribution. The wellknown relation

(18)

$$\chi = 4\pi \int_{T_{e}}^{1} k dT$$
 is valid

respectively with (5) and (7)

$$x_{1} \ln \frac{x_{2} + T_{cc}}{x_{2} + T_{f}} + \frac{x_{3}}{4} (T_{cc}^{4} - T_{f}^{4}) = \frac{\chi}{4\pi(1 - P_{o}^{3})}$$

b) $T_{cc} \ge T_{p1}$

Т

that means we get a hollow cylinder with a two zone porosity distribution. In (21) was derived the following relation

$$\int_{T_{f}}^{cc} k_{th} dT = \frac{\chi}{4\pi D_{s} (1 + \frac{\delta}{r_{f}})^{2}} \left\{ \frac{1 - P_{p1}}{1 - P_{p1}^{\frac{2}{3}}} \left[\left(\frac{r_{p1}}{r_{f}} \right)^{2} - \left(\frac{r_{cc}}{r_{f}} \right)^{2} - 2 \left(\frac{r_{cc}}{r_{f}} \right)^{2} \ln \frac{r_{p1}}{r_{cc}} \right] + \frac{1 - P_{o}}{1 - P_{o}^{\frac{2}{3}}} \left[1 - \left(\frac{r_{p1}}{r_{f}} \right)^{2} + 2 \left(\frac{D_{s} (1 + \frac{\delta}{r_{f}})^{2}}{1 - P_{o}} - 1 \right) \ln \frac{r_{f}}{r_{p1}} \right] \right\}$$
(19)

D = relative smear density

With the condition, that the central channel is provided by the porosity of the plastic region related to the smear density, that no gap is present and with (7), (11), (12) and (13) we obtain.

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$$\kappa_{1} \ln \left(\frac{\kappa_{2} + T_{cc}}{\kappa_{2} + T_{p1}} \right) + \left(\frac{\kappa_{3}}{4} (T_{cc}^{4} - T_{p1}^{4}) \right) = \frac{\chi}{4\pi} \frac{1}{1 - P_{p1}^{\frac{2}{3}}} \frac{V_{p1}}{V_{f}} \left[\frac{1 + \left(\frac{1 - P_{p1}}{D_{s}} - 1 \right) \ln \left(1 - \frac{D_{s}}{1 - P_{p1}} \right) \right] \quad (20)$$

2.3. Argumentation of the swelling behaviour

2.3.1. Swelling rates

Though integral swelling rates for a wide temperature range are known from literature, some detailed indications on gas- and solid swelling rates have supported the application of zone specific swelling rates (25, 26, 2). Solid swelling rates between 0.3 and 1.25 v/o/10 000 $\frac{MWG}{t \text{ heavy atoms}}$ depending on the degree of consumption of fissioned atom vacancy were published in (25). Gas swelling rate values between 0.55 and 0.8 MWd $v/o/10 000 \frac{1}{t \text{ heavy atoms}}$ dependent on used models werepublished in $\langle 26, 27 \rangle$. In addition we calculated such swelling rates. Swelling rates of both types were coordinated to the temperature zones by estimate in the following way. Regarding to the solid swelling rate the lattice vacancy of fissioned atoms will be widely consumed by the fission products at higher temperatures. At lower temperatures a smaller extent of lattice volume should be applicable because of lattice rigidity. Concerning the estimation of the gas swelling rate published results on fission gas release (28, 29) were taken into account. Furthermore the results of some calculations were included basing on following ideas:

Post irradiation examinations of high burnup specimens indicate that a certain amount of porous volume is existing at any time though remarkable external swelling has occured. These pores are non-growing and static gas pores as well as pores which are still under growth and migration.

Because of the uncertainty of data and models, which do not allow a quantitative classification of pores at present, a temperature, burnup and fission rate dependent amount of pores which is under remarkable high internal gas pressure is estimated as stabilized. They are of very small size $\langle 27 \rangle$, 10^{-5} to 10^{-6} cm in radius, if unmovable at medium or low temperatures. The main part of non-released fission gas seems to be trapped in such pores and in interstitial lattice positions. Hence the applied fission gas swelling rates could only be developed including fission gas release data. The following swelling rates are resulting:

	low temp. zone 1300 ⁰ C	creep zone 1300 - 1700 ⁰ C	plastic zone 1700 ⁰ C
Solid swelling rate	1	0,7	0,5
Gas swelling rate	0,6	1,0	0,2
Total swelling rate	1,6	1,7	0,7

Table I : Applied swelling rates for the temperature zones

The given swelling rate values are measured in v/o/10 000 $\frac{MWd}{t \text{ heavy atoms}}$

2.3.2. Availability Factors

The amount of void volume within the fuel which can be filled by the swelling process in practical case will not be completely consumable. Resulting from publications $\langle 30 \rangle$ fuel plasticity is dependent on fission influence. Even after fuel is under compression by fuel clad interaction voids of high resistance geometry will remain. Many postirradiation examination results show this. To be understood as averaged over the operation time, corresponding to burnup of 8 or 10 %, we apply factors which are representing the availability of voids related to as-fabricated porous volume, but not saying a not available void only can be of as-fabricated nature.

We get by estimate the following availability factors:

Table 2 :	Applied	availability	factors	for the	temperature	zones
	A A	2				

	low temp. zone	creep zone	plastic zone
	1300 ⁰ C	1300 - 1700 ⁰ C	1700 ⁰ C
availability factors	0,3	0,5	0,8

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These assumptions may be working pessimistic when applied; from design reasons this is intention. The influence of their eventual variation will be discussed later in chapter 3.3.2.

2.4. Qualitative discussion of the fuel-cladding interaction

The burnup model, formulated in chapter 2.1., assumes that the resulting maximum burnup will not be influenced by the mechanical fuel-cladding interaction. Furthermore we assume for the calculations in chapter 3, that the radial and axial deformations of the cladding are relatively small and that there is no axial swelling of the fuel. These boundary conditions require a special interaction behaviour. This comprehensive interaction problem, which still is unsolved, is only touched in this work.

2.4.1. Multiaxial stress and strain analysis of the creeping sheath

To initiate the treatment of the interaction problem multiaxial stress and strain analysis of the creeping sheath was carried out. If a symmetry of revolution and a plane state of strain is presumed the following equilibrium equation results:

$$\frac{d\sigma_{r}}{dr} + \frac{\sigma_{r} - \sigma_{\theta}}{r} = 0$$
 (21)

Index r, Θ , z indicate the radial, tangential and axial component of the stress σ and the strain ε

and the following compatibility equations:

$$\varepsilon_r = \frac{du}{dr}$$
 $\varepsilon = \frac{u}{r}$ $\varepsilon_z = constant$ (22)

 ε = (total) strain

u = radial deformation

Furthermore the following stress and strain relations were taken as a basis:

$$\varepsilon_{\mathbf{r}} = \frac{1}{E_{\mathrm{sh}}} \left[\sigma_{\mathbf{r}} - \mu_{\mathrm{sh}} (\sigma_{\theta} + \sigma_{z}) \right] + \alpha_{\mathrm{sh}}^{\mathrm{T}} \mathbf{sh} + \varepsilon_{\mathrm{rc}}$$

$$\varepsilon_{\theta} = \frac{1}{E_{\mathrm{sh}}} \left[\sigma_{\theta} - \mu_{\mathrm{sh}} (\sigma_{\mathbf{r}} + \sigma_{z}) \right] + \alpha_{\mathrm{sh}}^{\mathrm{T}} \mathbf{sh} + \varepsilon_{\theta c} \qquad (23)$$

$$\varepsilon_{z} = \frac{1}{E_{\mathrm{sh}}} \left[\sigma_{z} - \mu_{\mathrm{sh}} (\sigma_{z} + \sigma_{\theta}) \right] \neq \alpha_{\mathrm{sh}}^{\mathrm{T}} \mathbf{sh} + \varepsilon_{zc}$$

- E_{sh} = Young modulus of the sheath
- μ_{sh} = Poission ratio of the sheath

 ε_r , ε_{θ} , ε_z are the components of the total strains - ε_{rc} , $\varepsilon_{\theta c}$, ε_{zc} are the components of the strains due to the creeping.

The strains, given by the equations above, are distributed in elastic, thermal expansion and creep increments.

The stress and strain state of the sheath were determined by the following iterative method $\langle 31 \rangle$. Thereby is given the inner and outer pressure, the axial force, the temperature distribution and the creep law in the form of the Norton's creep law

$$\dot{\varepsilon}_{sh} = K\sigma^n$$
 (24)

 $\dot{\epsilon}_{eb}$ = Creep rate of the sheath

K,n = constants relating to the Norton's creep law

These input data respectively functions can be dependant on time. For the time t = 0, the creep strains are $\varepsilon_{rc} = \varepsilon_{\theta c} = \varepsilon_{zc} = 0$. The stresses and strains are calculated with (21), (22) and (23). From the components of the stress an equivalent stress is determined on the base of the von Mises theory and using the Norton creep relation (24), an equivalent creep strain is calculated for the following time interval. This creep strain is distributed in the three directions by the help of the von Mises theory. The creep strain components socobtained are now used as input functions of (23), so that new strain and stress values can now be calculated and the procedure begins again.

2.4.2. Axial expansion of the fuel

The three dimensional fuel swelling can both fill internal void volumes and occur externally in radial and axial direction. The internal void volume is essentially given by the hot smear density and the porosity availability factors. The external swelling is determined by external restraints and by the fuel-cladding interaction. There are only allowed small radial cladding deformations from reasons of coolant channel cross sections and brittleness of the cladding material.

The axial swelling has relevant consequences on the maximum operation time, e.g. an axial expansion of 4 v/o causes a rise of the maximum operation time of 25 % (fig. 6). Therefore the question arises by which factors the axial tolerances are limited. On the one hand this tolerance is given essentially by safety considerations. These must be oriented the Na2-core $\langle 32 \rangle$, which is taken as basis for all following calculations on two values: Firstly, a change of the fuel length of 1 cm at a fuellength of 100 cm, corresponding to an averaged value of thermal expansion causes about 2 \$ reactivity change in the core. Secondly, a temporal reactivity change of 2.5 \$/s is regarded as the maximum allowable limit for the core. Considering a sudden return movement of the axially expanded state to the original length, for which reasons ever, we see, that this allowed length change will mainly be consumed by thermal expansion. Therefore an axial swelling cannot be allowed in our calculations according to Na2. Apart from the fact, that this case is extremely pessimistic, the axial thermal expansion can be reduced by using dished pellets. On the other hand the long friction area could cause a not allowed axial deformation of the sheath, because of the interaction fuel cladding, if no axial restraint would be applied.

2.4.3. Rough estimate of the swelling pressure of the fuel

In the following an estimate of the swelling pressure of the fuel is made for strongly simplified assumptions. We consider the fuel swelling against the cladding which is assumed to be rigid. Then the radial fuel swelling velocity directed to the outside is equal to the creep rate of the fuel directed to the axis. According to the creep law, valid for the fuel, an annular tension σ is induced in a ring with radius r and the width dr. Corresponding to the pressure formula for thin walled tubes.

$$dp_{sw} = \sigma \frac{dr}{r}$$
 (25)

p_{sw} = swelling pressure σ = stress

a differential of the swelling pressure is resulting. The total swelling pressure is obtained by integration of (25) over the radius r. The pressure formula is only valid for thin walled tubes. But from the equality of (26)

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and (27) results a strong drop of the stress with rising temperature, so that this integration is justified.

The radial swelling velocity of the fuel is connected with the swelling rate under the assumption of a constant isotropic fuel swelling in the following way:

$$s = \frac{1}{3} \frac{A}{t} \cdot S$$
(26)
$$= \frac{1}{t} \frac{\Delta L}{L_{th}} = (linear) \text{ swelling velocity}$$

t = time

s

The ratio burnup divided by time A/t is given by (2) and proportional besides other factors to the linear rod power

The creep rate \mathcal{E}_{f} of the fuel is given by the publications of several authors $\langle 33 \text{ to } 36 \rangle$. These results can be represented in the following form according to $\langle 36 \rangle$.

$$\dot{\varepsilon}_{f} = e^{-\frac{Q}{RT}} (\omega_{1}\sigma + \omega_{2}\sigma^{\omega_{3}})$$
(27)

Q = activation energy

 ω_1 , ω_2 , ω_3 = constants relating to the creep law of the fuel R = gas constant

(26) and (27) is equalized and this equation is solved with respect to the stress σ . Then σ is substituted in (25) and the swelling pressure is obtained in dependance of the temperature distribution by integration of (25).

In the following the swelling pressure is estimated in an optimistic and pessimistic way. These estimations are represented in fig. 4. For the pessimistic estimate the experimental data of Armstrong $\langle 34 \rangle$ for stoichiometric UO₂ were used as the basis. $\omega_1 = 7.26 \times 10^6 \text{ cm}^2 \text{kp}^{-1} \text{h}^{-1}$, $\omega_2 = 2.22 \times 10^{-2} \text{ cm}^2 \text{kp}^{-1} \text{h}^{-1}$; Q = 91 kcal/mol, $\omega_3 = 4$. For the ranges of temperature and rod power of interest we obtain swelling pressures of 20 to 3 x 10³ kp/cm². According to this estimate, practically all zones of the Na2-core must be regarded as being imperiled, if it is assumed that the cladding, in addition to the fission gas pressure has to sustain about 100 kp/cm² swelling pressure.

For the optimistic estimate we use the creep data for $UO_{2.04}$ measured in $\langle 35 \rangle$. $w_1 = 1700 \text{ cm}^2/\text{kph}^4$, $w_2 = 5 \cdot 10^{-6} \text{ cm}^2 \text{kph}^4$, Q = 56 kcal/mol, $w_3 = 4$. With these data we obtain swelling pressures between 0.3 and 200 kp/cm² for the range of interest, and thus practically no zone in the Na2-core would have to be regarded as imperiled under the above assumption.

Even the creep analysis of irradiated specimens (30) do not allow accurate quantitative statements. However, according to these results the creep strength under irradiation decreases by some powers of ten (thermal spikes, increase in point defect concentration) so that the swelling pressure load on the cladding remains negligibly small in any case.

One comes to the conclusion that an estimate of the swelling pressure does not lead at present to satisfactory results. Furthermore it is not possible at present, to obtain a design criterion regarding to the maximum operation time on the base of the mechanical interaction fuel clad.

3. Maximum operation time computer analysis of the Na2-Core

3.1. General aspects concerning the dependences of the maximum operation time on core position and parameters

Parameter classification

The essential dependences of the maximum operation time on parameters are clearly represented in fig. 5 on the basis of (1) and (2).

Two types of parameters can be distinguished:

- I. Parameters, which directly influence the maximum operation time
 - a) the fuel radius r_f , given by the outer radius of the sheath, the initial gap width δ and the sheath thickness d_{sh}
 - b) the swelling rates S_{1t} , S_{cr} and S_{p1}
 - c) the availability factors G_{1t} , G_{cr} and G_{p1}
 - d) the initial porosity P
 - e) the linear rod power χ

- II. Parameters, which indirectly influence the maximum operation time over the volumes V_{1t}, V_{cr}, V_{p1} and V_{hg} given by the radial temperature distribution
 - a) the coolant temperature T_{Na}, given by the sodium inlet and outlet temperature and the linear rod power distribution
 - b) the heat transfer coefficient fuel clad h_{hg}
 - c) the zone boundary temperatures T_{cr} and T_{p1}
 - d) the porosity of the densified plastic zone P_{pl} and the central channel radius r_{cc}
 - e) the initial gap width δ
 - f) the thermal expansion coefficients of the fuel α_f and the sheath α_{sh}
 - g) the thermal conductivity of the theoretical dense fuel K_{H}
 - h) the initial porosity P
 - i) the linear rod power

The linear rod power and the initial porosity influence the maximum operation time both directly and indirectly over the radial temperature distribution.

The maximum burnup respectively the maximum operation time is only dependant on the core position with regard to the linear rod power and the coolant temperature.

One zone model

In the following we generally analyse the dependence of the maximum operation time on the core position. The clearest case is present, if the swelling rates and the availability factors in the three zones are equal, that means if it is realized a one zone model.

For the case of a one zone model it results from (1)

$$A_{\max} = \frac{P_o G + X}{(1 - P_o)S}$$
(28)

whereby

 $G = G_{lt} = G_{cr} = G_{pl} = availability factor for the one zone model$ $S = S_{lt} = S_{cr} = S_{pl} = swelling rate for the one zone model$ $X = \frac{V_{hg} + V_{ax}}{V_{f}} = relative hot gap and axial expansion volume ratio$

As explained in chapter 2.4.2. we use $V_{ax} = 0$ for our calculations. From the computer analculation for the Na2-core treated in the following chapter results that V_{hg}/V only changesabout ~20 % within the core. Because $X = V_{hg}/V$ is about 50 % of P_oG , the maximum burnup A_{max} can be regarded as constant in a first approach. According to (2) the maximum operation time is given by a $\frac{1}{\chi}$ - core position dependance. In fig. 6 A_{max} is represented graphically as a function of the relative fuel density $D_f = 1 - P_o$ with swelling rate, availability factor and relative hot gap and axial expansion volume ratio as parameters. The sensitive influence of these parameters on the maximum burnup can be seen. To make clear this sensitive parameter dependance the total differential of (28) is formed. With respect to $X \ll G$, it results

$$dA_{max} = -\frac{G}{S D_{f}^{2}} dD_{f} + \frac{1}{S} (\frac{1}{D_{f}} - 1) dG - \frac{(1 - D_{f})G + X}{D_{f}S^{2}} dS + \frac{1}{S D_{f}} dX$$
(29)

For the averaged values used in fig. 6

$$D_f = 0.85$$
 G = 0.5 X = 0.05 S = 1.6 \cdot 10^{-6} \frac{t}{MVd}

results from (29) that a burnup of 9360 MWd/t (heavy elements) corresponding to 1 % of heavy atoms is caused by the following parameter changes

$$dA_{max} = 9360 \frac{MWd}{t} \qquad \begin{cases} dD_{f} = -0,022 \\ dG = 0,086 \\ dX = 0,013 \\ dS = -0,163 \cdot 10^{-6} \frac{t}{MWd} \end{cases}$$

Three zone model

If the swelling rates and availability factors in the three zones are not constant, the $\frac{1}{\chi}$ - dependance of the maximum operation time is deformed by the core position dependance of the volumes V_{1t} , V_{cr} , V_{p1} and V_{hg} given by the temperature distribution.

It is possible to fix the following general statements. The burnup required to fill the available porous volume of a zone by its swelling is obtained by a simple consideration as follows

$$A_{i} = \frac{P_{o}}{1 - P_{o}} \frac{G_{i}}{S_{i}}$$
 $i = 1t, cr, p1$ (30)

Equal 30 shows that the ratio G/S of the concerning zone determines its maximum burnup.

The burnup necessary to fill the hot gap volume by simultaneous swelling of the three zones is given by

$$A_{hg} = \frac{V_{hg}}{(1-P_o)(S_{1t}V_{1t}+S_{cr}V_{cr}+S_{p1}V_{p1})}$$
(31)

This is valid because of the assumed free displaceability of the fuel. Three cases can easily be overlooked:

a) $\frac{G_1}{S_1} > \frac{G_2}{S_2} > \frac{G_3}{S_3}$ and max of $A_i > A_{hg}$ i = p1, cr or lt 2, 3 complementary

In this case the zone with the biggest value of $\frac{G_i}{S_i}$ determines the picture if this zone has a relatively big volume

b) $\frac{G_{p1}}{S_{p1}} \simeq \frac{G_{cr}}{S_{cr}} \simeq \frac{G_{lt}}{S_{lt}}$

That means that the single axial zone elements are filled by themselves in nearly the same time. Regarding the remaining filling of the hot gap volume the zone with the biggest swelling rate determines the picture if this zone has a relatively big volume.

c) $A_{hg} > max of A_i$

This is surely fulfilled if $V_{hg} > P_o V_f$. Here like case b) the zone with the biggest swelling rate determines the picture if this zone has a relatively big volume.

As it can be seen in chapter 3.3.1. case a) is realized for our data and conditions.

3.2. Computer listing of the maximum operation times for the Na2-core

In table 3 the input data for the computer program are compiled $\langle 37 \rangle$ These data are composed of the material data discussed in chapter 2 and the required Na2-data $\langle 32 \rangle$. The computer listing is represented in fig. 7. From left to

right the core radius is given and from above to below the axial core height. Relating to this diagram the sodium flows from above to below. In this two dimensional diagram the values represent the maximum operation times for the concerning radial and axial core positions. Instead of this two dimensional representation a curved area of the lifetimes extending over the R-Z-plane would be very clear.

As the Na-2 core is chaped like a vertical cylinder and since the distribution of fission rates and the temperature are in rotational symmetry to the cylinder axis, it is sufficient to know the lifetimes for the rotational area. The rotational area is formed by the core height of 96 cm and the core radius of 77 cm. Radially, the core is subdivided into two fuel zones of different degrees of enrichment. The fuel zones can be subdivided into idealized annular subassembly (four on the inside, two on the outside).

For better clarity, the axial lifetime profiles for 5 radial positions, a line connecting the points of minimum lifetime and lines of equal lifetime (450 d, 500 d) where plotted in the computer listing.

The axial lifetime profiles exhibit a relatively flat curve over a wide internal core range. However the boundary zones (mantle, top and bottom regions) clearly exhibit increasing lifetimes. Quantitively, for the fraction of the core volume with lifetimes longer than 500 d, the results some 40 vol.% (zone I 25 vol.%, zone II 60 vol.%).

The minimum line indicates the points of the minima of the axial lifetime profiles. The slightly curved decreasing slope of the minimum line can be seen. The absolute minimum localized at (R,Z) = (9, 56), amounts to 397 d. With increasing distance from the center axis of the core the values of the minimum line increase only a little i.e. about (+7%). Only at the very edge of the core $R \ge 74$ cm there is a steep increase which is proportional to the radial power decrease in this core area, because there exists only one zone, so that the influence of the temperature distribution is negligible.

In the inner core region the subassembly effect becomes valid. This is evident from the lines of equal lifetimes which show a little jump at the boundary between subassemblies. Subassembly effect, in our terminology, means the effect that a constant throughput of coolant within one subassembly results in a temperature jump of the coolant between two adjacent subassemblies as a consequence of the radial power gradient. It does not appear at all points where the maximum temperature is smaller than 1300°C, so that only one zone is existing.

In fig. 8 the maximum burnup is represented in dependance on smear density with linear rod power and cladding temperature as parameters on the base of the required data, compiled in fig. 7. This picture shows the influence of the temperature distribution on the maximum burnup if a three zone model is used.

3.3. Discussion of the results

3.3.1. Parameter analysis

In this chapter the results of the computer listing are discussed especially the axial lifetime profiles and the minimum line on the base of the given data with the help of the considerations of chapter 3.1. Chapter 3.1. shows that in the case of a three zone model the maximum operation times are relevantly dependant on the ratios G_i/S_i . In table 4 these ratios and the corresponding burnups required to fill the single zones are represented. Furthermore the burnup required to fill the hot gap volume by simultaneous swelling of the fuel zones according to (31) is included. Table 4 : Applied ratios $\frac{G_i}{S_i}$ and corresponding burnups A_i and A_{hg} $\left[P_o = 0,155/\frac{V_{hg}}{V_e} = 0,045/\overline{S} = 1,4\cdot10^{-6} \frac{t}{MVd}\right]$

zone i	Gi	$B_{i}\left[10^{-6}\frac{t}{MVd}\right]$	$\frac{G_{i}}{S_{i}} \left[10^{6} \frac{MWd}{t} \right]$	$A_{i} \left[10^{4} \frac{MWd}{t} \right]$
pl	0,8	0,7	1,14	20,9
cr	0,5	1,7	0,29	5,5
lt	0,3	1,6	0,19	3,5
hg	-	-		3,8

It results that A_{pl} is relevantly bigger than A_{cr} and A_{lt} as well as A_{hg} . Consequently the case a) of chapter 3.1. is realized so that the plastic region is prevailing if its volume is comparable with the low temperature and creep zone volume.

In fig. 9b the axial dependance of the temperature volume ratios and the resulting axial lifetime profile is represented for a position near the core axis. The axial profile of the volume ratios can be understood with the help of the equations (11), (12), (13). If the maximum temperature is smaller than $T_{cr} = 1300^{\circ}$ C, there is only one temperature zone. If the maximum temperature is bigger than 1300° C and smaller than 1700° C a two zone model is realized. $\frac{V_{1t}}{V_f}$ is approximately proportional to $\frac{1}{\chi}$. This is shown in fig. 9 for a cosine shaped power curve. $\frac{vcr}{V_f}$ is then complementary to 1. If the maximum temperature is bigger than $T_{p1} = 1700^{\circ}$ C a three zone model is v_{cr} is now exactly proportional to $\frac{1}{\chi}$. $\frac{v1}{V_f}$ is complementary to 1. In all cases the nonsymmetric curve of the sodium temperature relating to the central plane must be regarded. Therefore the low temperature volumes are relatively bigger in the colder half of the core for positions axialsymmetric to the central plane.

Fig. 9a shows that in the outer regions the $\frac{1}{\chi}$ - dependance is realized according to the one zone model if the maximum temperature is lower than 1300° C. This curve is only deformed a bit if the creep zone is existent because of the relatively small differences of the G/S ratios. A relevant flattening of the lifetime profile occurs if the plastic zone is existent due to the approximately contrary effects of linear rod power and the plastic volume.

The position of the minimum of the lifetime coincides with the onset of the plastic zone in the colder half of the core, because the plastic zone has a smaller axial extension measured from central plane in this core half than in the hotter one. In fig. 9a the thin drawn line representates the one zone model if the datas of the low temperature zone are used. The closely dotted line corresponds with a two zone model whereby the low temperature zone data are attributed to the plastic zone, too. The thick line representates completely the three zone model. Fig. 10 shows how the shape of the axial lifetime profiles is strongly influenced by the grading of the swelling rates and availability factors among each other.

The conclusion can be drawn that if the ratios $\frac{G_i}{S_i}$ are fixed an increase of the lifetime is either possible by a decrease of the plasticity temperature T_{pl} or by an increase of the temperature profile An increase of the temperature profile can be reached by a higher sodium temperature, a smaller hot gap heat transfer coefficient and a smaller thermal conductivity of the theoretical dense fuel. The effect of the fuel porosity and the linear rod power which influence the maximum operation time both directly and indirectly over the temperature distribution is calculated in the following chapter. This increase of lifetime is connected with a shifting of the position of the minimum of the axial lifetime profile in direction of the sodium inlet because this position is given by the onset of the plastic zone. The value of this minimum lifetime is naturally independent on the swelling rate and availability factor of the plastic zone.

Furthermore the lifetime diagram is relevantly influenced by variations of availability factor G and swelling rate S, especially of the ratios $\frac{G}{S}$ among each other. These effects are calculated in the following chapter.

3.3.2. Error analysis

Table 5 indicates the influence of the errors of the important parameters. The errors were evaluated by the reactor design and by irradiation experiments. Lifetime diagrams were calculated for the input data represented in table 3 and the following parameter variations.

- a) linear rod power and initial fuel porosity, which influence the maximum operation time both directly and indirectly over the temperature distribution.
- B) the swelling rates and the availability factors of the three zones whereby the grading of these parameters between each other are kept approximately constant.
- γ) fuel thermal conductivity, heat transfer coefficient fuel clad, zone boundary temperatures and sodium temperature increase which influence the maximum operation time only indirectly over the temperature distribution.

A part of the computer listings is represented at fig. 11 to fig. 15. The computer diagrams are described in table 5 by the following three values:

- a) value and position of the absolute minimum of the lifetime
- b) shift of the minimum line (in the direction of the center of the core (+), in the direction of coolant inlet (-).
- c) position and value of the minimum lifetime in the center plane of the core

The following results are obtained:

- a) The absolute minimum of the lifetime is not influenced by the range of variation of the swelling rate and the availability factor of the plastic zone. The influence remains relatively small with the parameters of rod power and sodium temperature increase. The absolute minimum of the lifetime reacts very sensitive to the two parameters of fuel porosity and swelling rate in the low temperature range. The influence of the heat transfer coefficient fuel clad remains relatively small.
- b) As we have seen in the preceding chapter the absolute minimum of the lifetime is located at the onset of the plastic zone in the colder half of the core because the ratio $\frac{G}{S}$ is relevantly larger in the plastic zone than in the outer zones. Therefore the availability factors, the swelling rates and the creep temperature (1300°C) have no influence on this position. The parameters which influence the temperature distribution cause a shifting of the minimum line, only the effect of the sodium temperature increase is negligible because the minimum line is located at the colder half of the core.
- c) The minimum of the lifetime in the core center plane is at the edge of the core, with one exception, and it is about 10 % higher than the absolute minimum, which must always be sought near the core axis.

The algebraic sum of all the single errors shown in table 5 is $\frac{+48,2\%}{-58,1\%}$ for the absolute minimum of lifetime. However, if all the pessimistic and optimistic values (with respect to the lifetime) are fed into the program simultaneously the devation from the standard value for the absolute minimum of the lifetime is $\frac{+66\%}{-36\%}$. As estimates of this type are not very realistic, the mean square error $\sqrt{\sum_{i=1}^{n} \Delta x_{i}^{2}}$ for the single errors Δx_{i} shown in table 5 is calculated to be $\pm 20\%$.

4. Summary

The aim of this work was the calculation and analysis of the maximum operation time respectively of the maximum burnup of the fuel pins of a core as a function of the distance from the core centerline and core midplane.

As the basis of this examination there was taken a defined three zone volume balance relating to the swelling of the fuel volume and the consumption of void volume, given by the hot gap volume, the axial expansion and ratios of the pore volume. To calculate the maximum burnup with this model the radial temperature distribution had to be treated quantitatively. For doing this a new porosity relationship of the thermal conductivity was used, which was developed recently. Furthermore a two zone porosity model was applied for the characterization of the porosity migration in the plastic region connected with the central channel formation and a densification in the plastic region. ("Thermal central channel effect"). To get the hot gap volume it was necessary to calculate the radial thermal expansion of the fuel at fixed porosity on the base of a plastic behaviour. The applied swelling rates and availability factors were argued. The still unsolved fuel clad interaction problem was discussed. To initiate the quantitative treatment of the interaction problem a multiaxial stress and strain calculation of the creeping sheath was carried out by an iterative method. It was founded that no axial swelling was allowed and the swelling pressure of the fuel was roughly estimated.

After having discussed generally the dependance of the maximum operation time on core position and parameters the lifetime was calculated as a function of the position within the core for the case of the Na2-Core. We roughly got constant maximum operation times in radial direction, in axial direction we also have constant values in a wide range, but towards the core edge we have a strong rise of the maximum operation times. We made extensive parameter studies, to test the sensitivity of the maximum operation times with respect to the influence of temperature distribution, swelling rates , availability factors etc. This parameter study is especially important with respect to the grade of accuracy applied on fuel pin specific data like swelling rates, availability factors, plastic and creep data, coefficients of the thermal conductivity equation

, thermal expansion coefficients and heat transfer coefficients By this procedure we can check the range of the validity of the results and get a general survey on fuel pin behaviour under operational conditions.

Nomenclature

A	=	burnup
С	=	constants relating to pore conductivity
D	=	relative fuel density
đ	=	length of pore in direction of heat flow, thickness
Ε	=	young modulus
G	=	availability factor
h	8	heat transfer coefficient
К	B	constant relating to Norton's creep law
k	22	thermal conductivity
Ļ	-	length
n	=	constant relating to Norton's creep law
Р	=	porosity
р	-	pressure
Q	=	activation energy
R	*	gas constant
r	=	radius
S	8	swelling rate
S	=	swelling velocity
Т	8	temperature
t		time
u	*	radial deformation
V	=	Volume
z	#	axial direction
α	*	thermal expansion coefficient
^a i	æ	constants relating to thermal expansion
ß	82	shape factor
δ		initial gap width
ε	2	strain
ŋ	#	constant relating to linear porosity relation
0	8	tangential
×.	*	constants relating to thermal conductivity
μ	53	Poission ratio
ρ	2	(absolute) density

σ	=	stress
χ	=	linear rod power
ωi	-	constants relating to creep law of UO_2
Ind	ices	
ax	=	axial
cc	z	central channel
cr	**	creep
cond	d =	conduction
f	=	fuel, fuel surface
h	=	heavy elements
hg	=	hot gap
1t	22	low temperature
max	=	maximum
Na	=	coolant sodium
р	=	pore
p1	æ	plastic
r	=	temperature radiation
S	11	smear
sh	=	sheath (cladding)
sw		swelling
th	*	theoretical

- 28 -

o =

initial

1,2,3 = distinction of constants

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MATERIAL DATA		REACTOR DATA:	
INITIAL GAP WIDTH	6 = 70 μ	LINEAR ROD POWER	X max = 420 W/cm
INITIAL POROSITY	P _o = 0.155	Na INLET TEMPERATURE	T _{Na,in} = 385 ° C
POROSITY IN THE PLASTIC ZONE	P _{pl} = 0.05	Na OUTLET TEMPERATURE	T _{Na,out} = 585 ° C
HEAT TRANSFER COEFF. FUEL-SHEATH	$h_{hg} = 1.1 \frac{W}{cm^2 \circ C}$	HEAT TRANSFER COEFF. SHEATH - Na	$h_{Na} = 14 \frac{W}{cm^2 \circ C}$
THEOR. THERM. CONDUCTIVITY FUEL	$K_{\text{th,f}} = \left(\frac{41.2}{T+4.9} + 6.55 \cdot 10^{-13} \text{ T}^3\right) \frac{W}{\text{cm}^\circ \text{K}}$	OUTER SHEATH RADIUS	r _{sh,out} = 0.3 cm
THERMAL CONDUCTIVITY SHEATH	K _{sh} = 0.21 W Tin ^o K	INNER SHEATH RADIUS	r _{sh,in} = 0.262 cm
THERM. EXPANSION COEFF. FUEL	$\alpha_{f} = (3.9 \cdot 10^{-9} \text{ T} + 7.87 \cdot 10^{-6}) \frac{1}{\circ \text{C}}$	SMEAR DENSITY	D _s = 0.80
THERM. EXPANSION COEFF. SHEATH	$\alpha_{sh} = 1.75 \cdot 10^{-5} \frac{1}{\circ C}$ T in °C	AXIAL SWELLING VOL.	$V_{ax} = 0$
CREEP TEMPERATURE	T _{cr} = 1300 °C	CORE RADIUS	R = 76.5 cm
PLASTIC TEMPERATURE	T _{pl} = 1700 °C	CORE HEIGHT	Z = 95 cm
SWELLING RATES	$S_{lt} = 1.6 \cdot 10^{-6} \frac{t}{MWd}$	MESH DATA:	
FOR THE THREE	$S_{cr} = 1.7 \cdot 10^{-6} \frac{t}{MWd}$	RADIAL STEP WIDTH	ΔR = 2.41 cm
TEMPERATURE ZONES	$S_{pl} = 0.7 \cdot 10^{-6} \frac{t}{MWd}$	NUMBER OF RADIAL POINTS	30
AVAILABILITY FACTORS	G _{lt} = 0.3	AXIAL STEP WIDTH	ΔZ = 2.0 cm
FOR THE THREE	G _{cr} = 0.5	NUMBER OF AXIAL POINTS	49
TEMPERATURE ZONES	G _{pl} = 0.8	TABLE 3 INPUT	- DATA

Parameter variation				Absoluthe li	ute minim lfe time	um of	Shift: ing of	Mini life	mum (time	of the in the
Parameter		normal value	Error	value (d)	position (R/Z)	deviat (%)	the min line (cm)	cent: val. (d)	ral pos R/Z	plane . deviat (7)
linear rod power $y(R/Z)$ $(\frac{W}{2})$	•		 → (+20%) → (-20%) 	392 413	7/52 7/70	- 1,2 +5	-8 +14	453 441	7 33	+2,3 -1
initial porosity			0 175/+127)	441	7/5/	<u></u>		400	57	412.6
P_0 (1),		0,155	0,135(-13%) 0,135(-13%)	335	7/58	-15,6	+ 2	363	52 52	-16,6
			0,9(+28%)	395	12/56	~́		433	52	-0.6
swelling	S _{p1}	0,7	0,5(-28%)	395	12/56	0	-	438	52	+0.6
			1.9(+12%)	381	7/56	-3,5	-	419	52	-4
rates	Scr	1,7	1,5(-12%)	411	9/56	+4	-	454	52	+4
$s_i \left(\frac{V/0}{10} \frac{10}{t(U+Pu)} \right)$			1,8(+13%)	364	7/56	-7,4	-	403	52	-7,6
	^S lt	1,6	1,4(-13%)	432	7/56	+9,4	-	473	52	÷8,5
max(S _{pl} ,S _{cr}	,S ₁₊)		0,9;1,9;1,8	352	7/56	-10,9	-	387	52	-11,7
min(S _{p1} ,S _{cr}	, ^S 1t)		0,5;1,5;1,4	451	7/56	+14,2	-	496	52	+13,8
			0,88(+10%)	397	12/56	0	-	446	52	+0,8
availability	^G pl	0,8	0,72(-10%)	397	12/56	0	-	439	52	-0,8
factors	<u> </u>		0,55(+10%)	406	9/56	+2,4	-	453	52	+2,3
	Ger	0,5	0,45(-10%)	387	9/56	-2,4	-	433	52	-2,3
G ₁ (1)	_		0,33(+10%)	419	9/56	+3,2	-	455	52	+2,8
	Glt	0,3	0,27(-10%)	384	9/56	-3,2		431	52	-2,8
max(G _{p1} ,G _{c1}	,G _{1t})	0,88	3;0,55;0,33	423	12/58	+6,5	-	469	52	+5,9
min(G _{p1} ,G _{c1}	,G _{1t})	0,72	2;0,45;0,27	371	12/58	-6,5	-	417	52	-5,9
tono horm-	<i>6</i> 0		- 1750(+3%)	386	9/58	-2,3	+2	417	52	-4,3
zone boun-	¹ p1	1700	- 1650(-3%)	410	7/54	+3,8	-2	455	52	+4,3
dary temperatures	-		- 1350(+4%)	391	7/56	-1	· -	431	52	-1
T. (^o C) ——	^T cr	1300	- 1250(-4%)	399	7/56	+1	-	440	52	+1
i max(T	,T _{cr})		750 ,1350	382	7/56	-3,3	+2	412	52	-5,3
min(T _{pl}	, ^T cr)		650, 1250	414	7/54	+4,8	-2	459	52	+5,3
thermal conductivit	y fuel	ų	- (+10%)	369	7/60	-6,6	+4	410	52	-6
k (W/cm [°] C)			← (-10%)	427	7/52	+8,1	-4	485	52	+11,?
heat trans. coeffic	ient		1,5(+36%)	382	9/59	-3,8	+3	415	52	-4,8
fuel clad h _{hg} (W/cm	°°C)	¹ , ¹	0,7(-36%)	431	12/5?	+8,3	-4	503	52	+13,5
coolant temp. incre	ase		265(+43%)	402	12/56	+1,2	-	466	52	+5,2
AT _{Na} (°C)		185	135(-27%)	393	7/56	-1,2	-	420	52	-5,2

Table 5: Error analysis of the maximum operation time

Dependence of the porosity factor $1 - P^{2/3}$ Fig.1 and of the Loeb straight lines on the porosity

Porosity - Relation $K(T,P) = K_{th}(T) [1 - P^2/3]$

Loeb straight lines

 $K(T, P) = K_{th}(T) [1 - \eta P]$



Fig. 2 Thermal conductivity K for stoichiometric UO_2 and $(U_{0,8} Pu_{0,2})O_2$ in dependence of the temperature T and porosity P as parameter.

$$K(T, P) = K_{th}(T) \left[1 - P^{2/3} \right]$$

$$K_{th}(T) = \frac{41.2}{T + 4.9} + 6.55 \cdot 10^{-13} T^{3} \pm 7\% \left[\frac{W}{cm \circ K} \right] T in \circ K$$



FIG. 4

SWELLING PRESSURE OF THE FUEL IN DEPENDENCE OF THE LINEAR ROD POWER WITH INNER CLADDING TEMPERATURE $T_{sh,in}$ AS PARAMETER.



.

Fig. 5 (Essential) parameter dependences of the maximum operation time t_{max}.

Symbols :



Parameter variation of the burnup-formula Fig. 6 for the one-zone - model.



	RADIUS	Z	ONE 1	ZONE 2
BOTTOM	$\frac{Z}{R} = \frac{7}{12} \frac{5}{12} \frac{12}{14} \frac{16}{19} \frac{19}{21} \frac{21}{16}$	24 <u>26 28 21 33 26 38 4</u>	<u>43 45 48 5, 52</u>	55 57 6 62 65 67 69 72 74 77
	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$. 165 189 815 842 871 7 2 724 746 773 8).	631 655 681 71, 745 783 843 9181.461111 577 599 623 650 682 717 772 841 921018
	$\begin{array}{c} \begin{array}{c} \begin{array}{c} \begin{array}{c} \begin{array}{c} \begin{array}{c} \begin{array}{c} \end{array} \\ \end{array} \\ \end{array} \\ \begin{array}{c} \end{array} \\ \end{array} \\ \begin{array}{c} \begin{array}{c} \end{array} \\ \end{array} \\ \end{array} \\ \end{array} \\ \begin{array}{c} \end{array} \\ \end{array} \\ \begin{array}{c} \end{array} \\ \end{array} \\ \end{array} \\ \begin{array}{c} \end{array} \\ \end{array} \\ \end{array} \\ \begin{array}{c} \end{array} \\ \end{array} \\ \end{array} \\ \end{array} \\ \end{array} \\ \end{array} \\ \begin{array}{c} \end{array} \\ \end{array} $	2 324 531 538 550 559 57 586	64 623 643 665 689 6 4 623 643 665 689	531 575 596 629 661 713 777 852 941 494 572 533 556 584 614 563 723 793 876
	$\begin{array}{c c} \mathbf{\hat{H}} & \mathbf{\hat{H}} \\ \mathbf{\hat{H}} \\ \mathbf{\hat{H}} & \mathbf{\hat{H}} \\ \mathbf{\hat{H}}$	7 472 476 481 400 475 482	532 548 566 586 6 7	448 442 476 49 50 514 519 582 536 599 773
	54 41. 412 414 417 42; 423 427 4	3 434 437 444 45 457 465 45 4 419 422 420 42	486 497 5 8 526 542	41 424 436 453 471 43 52 571 628 698
	58 47.416 4 5 4 4 432 4 2 4 5	4 410 525 420 434 441 448 438 4 4 3 413 41° 426 434 443	469 480 491 4 52 3 454 465 476 489 5 2	4 7 4 4 4 4 4 1 4 36 4 54 4 4 73 5 1 5 4 5 9 9 6 8 4 4 7 4 4 6 4 2 1 4 3 8 4 5 7 4 8 6 5 2 1 5 7 4 6 3 6
	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	4 413 411 41, 4,7 4 4 4 4 4 8	44. 451 462 475 489 3 428 439 45 463 476	414 419 4 3 4 7 424 444 472 5 5 511 612 421 4 6 411 4 412 431 459 498 553 591
	66 431 43, 43 429 428 428 427 4 66 431 43, 43 429 428 428 427 4	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	417 427 438 451 465	425 412 418 412 44 42° 447 48° 516 72 431 428 423 418 411 41 437 47° 36 555
	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	416 412 419 432 446 3 423 418 413 424 438	439 486 433 429 423 422 2 453 489 528
	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	N. 439 438 438 437 434 432 425 3 442 441 442 441 439 437 433	429 424 419 41 431 3 434 430 425 419 425	442 440 437 433 428 428 416 446 482 521 445 443 441 437 432 433 421 44 476 515
	70 448 442 443 448 447 448 447 4 78 451	7 446 445 446 445 443 441 438 5, 449 449 455 449 447 445 442	439 435 430 424 420 443 439 435 429 423	448 447 444 441 436 438 426 435 471 10 451 450 448 445 440 443 431 481 467 9,7
	82 497 457 457 456 455 455 457 457 4 82 497 457 457 456 457 457 4	6 456 455 457 455 454 452 466 6 456 455 457 455 454 452 449	5 448 444 441 434 428 5 452 448 444 438 432	454 453 451 448 443 447 436 428 464 504 457 456 454 451 447 451 44, 420 462 5 2
MIDDLE		3 462 461 463 467 457 457 455 453	3 455 452 448 442 436/ 459 456 451 446 439	460 439 457 454 450 455 444 429 461 501 463 462 460 457 453 459 448 433 461 501
LINC	9) 469 466 463 463 465 466 466 466 4 9) 469 465 468 468 469 469 469 469 469 469 469 469 469 469	96 465 464 467 465 464 462 459 99 468 467 479 468 467 465 462	0 463 459 455 449 443. 2 466 462 458 452 446	466 465 463 460 456 462 452 436 462 562 469 468 466 463 459 466 455 44, 464 5.4
	94 415 475 475 474 474 476 475 4 94 475 475 476 476 476 476 476 476	2 471 475 473 472 475 488 465 5 474 473 476 475 473 471 468	3 469 466 461 455 449 3 472 469 464 458 451	472 411 469 466 461 469 458 441 466 557 475 414 472 469 464 472 461 445 470 111
	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	$\begin{array}{c} 1 \\ 1 \\ 46 \\ 479 \\ 482 \\ 481 \\ 479 \\ 477 \\ 477 \\ 473 \\ 477 \\ 473 \\ 477 \\ 473 \\ 477 \\ 473 \\ 477 \\ 473 \\ 477 \\ 473 \\ 477 \\ 473 \\ 477 \\ 473 \\ 475 \\ 475 $	475 471 467 460 454 3 478 474 469 462 456	478 477 475 471 467 475 463 447 475 516 482 48 477 474 469 478 465 448 482 522
	102 468 488 468 467 487 489 489 488 4 102 468 488 468 467 487 489 489 481 48	485 485 485 484 481 479 48 7 486 485 488 486 484 481 478	5 481 476 471 464 458 5 483 478 473 466 459	485 483 480 476 471 480 467 452 489 530 489 486 483 479 473 483 468 460 497 539
	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	3 491 492 493 491 489 488 485 481 5 491 492 493 491 488 485 481	. 485 480 474 468 457 . 486 481 476 468 464	492 489 485 481 474 484 471 470 57 560
	$\begin{array}{c} 100 \\ 110 \\ 110 \\ 112 \\$	19 484 492 495 495 495 496 486 482 17 495 493 497 494 491 487 482	2 487 482 476 468 477 2 488 483 476 476 49 488 483 476 476 49	458 44 493 484 476 487 471 49 581 573 531 466 491 484 476 488 472 57 558 588
	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	9 497 494 498 495 491 487 481 9 497 494 498 494 491.487 481	487 481 492 5 8 523	5.5.49 492 484 476 488 487 523 62 605 5.5 499 492 483 474 486 54 54 580 528
	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	9 496 497 497 495 490 485 4 9 496 493 497 493 487 485 44 7 497 497 497 493 487 485 44	409 512 527 542 510 521 533 546 564	5.5 47 493 485 40 14 34 582 62 68
	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	1 405 49 495 500 525 533 54 1 405 51 50 525 533 54 322 500 526 526 525 533 54	559 571 583 599 615	5 3 476 486 497 52 538 569 6.7 653 722 5 2 46 5 5 1 518 5250 65 597 634 69. 763
	$\begin{array}{cccccccccccccccccccccccccccccccccccc$	527 527 526 550 55 502 5 27 553 559 568 576 585 593 6 5 87 564 663 611 625 639 64	600 030 013 029 045 601 633 646 662 686 650 670 695 711 739	491 512 529 546 595 595 521 669 184 811 528 544 565 578 598 629 661 716 784 864
	13 0 566 385 593 597 662 649 614 6 13 2 667 63 635 647 645 652 658	9 326 633 643 652 661 671 623 3 671 678 689 689 714 729 747	3 7 2 72 743 770 80°	6 3 62 67 655 682 716 769 835 9 314
TOD	134 615 679 83 688 694 713 711 7 134 79 746 75 762 773 703 701	732 74 757 772 787 8 4 824	847 87. 394 928 963	7:1 729 758 79 826 867 9291 71. 913 901.95
IOP		0 600 700 800 400 500	600 700 800 900	400 500 500 700 800 400 500 500 800 900 10(
	500 50	JU 500	0	- 500 - 500
•			min of	operation time
	Profiles of axial operatio	n time l		th const. operation time [d]
FIG	1.7 COMPUTER L	ISTING OF TH	IE MAXIN	<i>IUM OPERATION TIMES</i>
	OF THE Na2-0	CORE		

FIG.8 DEPENDENCE OF THE MAXIMUM BURNUP ON THE SMEAR DENSITY WITH LINEAR ROD POWER AND INNER CLAD TEMPERATURE AS PARAMETERS.

CALCULATED ON THE BASE OF THE MATERIAL- AND No.2 - DATA COMPILED IN TABLE 3



FIG.9 INTERPRETATION OF THE AXIAL PROFILES OF THE MAXIMUM OPERATION TIMES BY THE RATIOS $\frac{G}{S}$ AND THE AXIAL PROFILES OF THE TEMPERATURE VOLUMES.

ZONE	Gi	$S_{i} \left[10^{-6} \frac{t}{MWd} \right]$	$\frac{G_{i}}{S_{i}} \left[10^{6} \frac{MWd}{t} \right]$
lt	0.3	1.6	0.19
cr	0.5	1.7	0.29
pl	0.8	0.7	1.14

Calculated on the base of the material – and Na 2 – data compiled in table 3 for a position near the core axis.



FIG.10 THE INFLUENCE OF THE ZONE SWELLING RATES AND THE AVAILABILITY FACTORS ON THE AXIAL MAXIMUM OPERATION TIME PROFILES.

CASE	LOW TEMPERATURE ZONE			CREEP ZONE			PLASTIC ZONE		
	G	S	e S	G	S	<u>e</u>	G	S	olo
1= normal	0.3	1.6	0,19	0.5	1.7	022	0.8	0.7	1.14
2	0.3	1.6	0 19	0.4	2.0	0.2	0.8	0.7	1.14
3	0.3	1.6	019	0.6	1.4	0.43	0.8	0.7	1,14
4	0.3	1.6	019	0.5	1.7	0 29	0.6	1.1	0.55
5	0.3	1.6	019	0.5	1.7	0.29	0.6	1.5	0.4

CALCULATED ON THE BASE OF THE MATERIAL - AND No.2 - DATA COMPILED IN TABLE 3 FOR A POSITION NEAR THE CORE AXIS

G = AVAILABILITY FACTOR

S = SWELLING RATE $\left[10^{-6} \frac{t}{MWd}\right]$

INFLUENCE OF THE CREEP ZONE

INFLUENCE OF THE PLASTIC ZONE





FIG.11 OPERATION TIME ANALYSIS IN Na 2 - CORE, INFLUENCE OF PARAMETERS CHANGED VALUE OF PARAMETER : ROD POWER



CHANGED VALUE OF PARAMETER: FABRICATED FUEL POROSITY



FIG.13 OPERATION TIME ANALYSIS IN Na-2 CORE, INFLUENCE OF PARAMETERS CHANGED VALUE OF PARAMETER : SWELLING RATE LOW TEMP. ZONE



FIG.14 OPERATION TIME ANALYSIS IN Na2-CORE, INFLUENC OF PARAMETERS CHANGED VALUE OF PARAMETER: TRANSFER COEFF. FUEL-CLAD



FIG.15 OPERATION TIME ANALYSIS IN Na 2-CORE, INFLUENCE OF PARAMETERS CHANGED VALUE OF PARAMETER: OUTLET TEMPERATURE Na